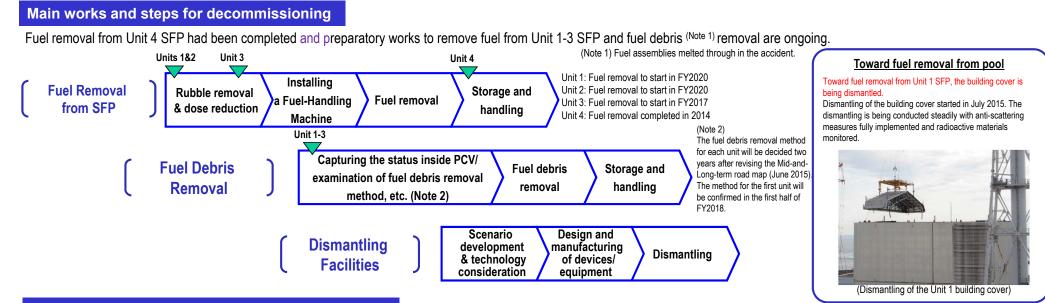
Summary of Decommissioning and Contaminated Water Management November 26, 2015

Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment

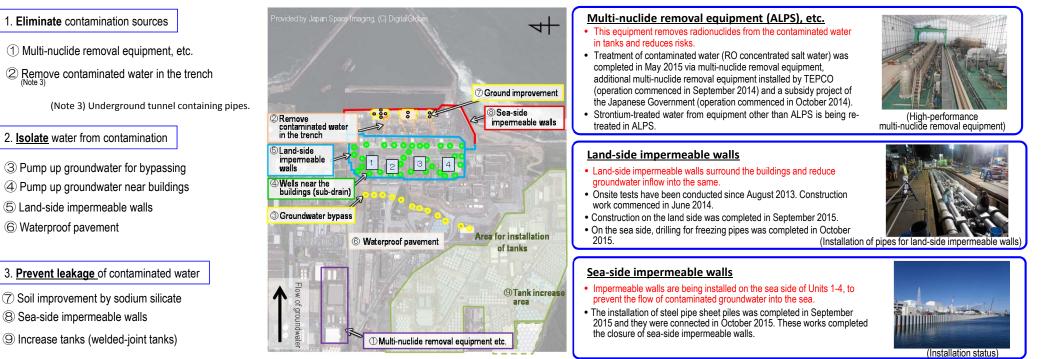


Three principles behind contaminated water countermeasures

6 Waterproof pavement

8 Sea-side impermeable walls

Countermeasures for contaminated water are implemented in accordance with the following three principles:



Progress Status and Future Challenges of the Mid- and Long-Term Roadmap toward Decommissioning of TEPCO's Fukushima Daiichi Nuclear Power Station Units 1-4 (Outline)

Progress status

The temperatures of the Reactor Pressure Vessel (RPV) and the Primary Containment Vessel (PCV) of Units 1-3 have been maintained within the range of approx. 20-40°C^{*1} for the past month. There was no significant change in the density of radioactive materials newly released from Reactor Buildings in the air¹². It was evaluated that the comprehensive cold shutdown condition had been maintained.

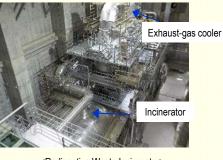
* 1 The values vary somewhat depending on the unit and location of the thermometer.

* 2 In October 2015, the radiation exposure dose due to the release of radioactive materials from the Unit 1-4 Reactor Buildings was evaluated as less than 0.0019 mSv/year at the site boundaries. The annual radiation dose by natural radiation is approx. 2.1 mSv/year (average in Japan).

Test operation of Radioactive Waste Incinerator started

Regarding the Radioactive Waste Incinerator, which will incinerate used protective clothing temporarily stored on site, installation of the facility was completed. Exhaust gas generated from incineration will be released after removing radioactive materials.

An incineration test using dummy waste started from November 25. Operations will start within this fiscal year.



<Radioactive Waste Incinerator>

Results of questionnaire survey for workers to improve the labor environment

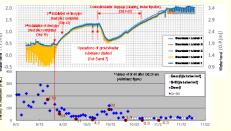
To improve the labor environment of workers at the power station, a 6th questionnaire survey was conducted, to which approx. 90% of workers responded.

Approx. 80% of respondents rated the following items as "good" or "reasonably good": operation start of the large rest house and dining room and expansion of the area where wearing of full-face masks is not required. Improvements will be made to meet requests such as setting up parking and rest houses on and off-site, and installing shower facilities.

Building cover Blowout panel Cover for fuel removal closed) Reactor Building (R/B) Removed fuel (assemblies) Spent Fuel Pool **1533**/1533 on mountain Primarv side noval completed on December 22, 2014 ontainment Vesse (PCV) Water 566 Reactor Pressur Vessel (RPV) Vent pipe Torus room Suppression - Chamber (S/C) Unit 3 Unit 4 Unit 1 Unit 2 removed first in 2012 Monitoring of seawater before and after Land-side impermeable walls

closure of sea-side impermeable walls

Since the closure of the sea-side impermeable walls on October 26, a steady decrease has been identified in the density of radioactive materials inside the port. During this stage, when the effect of the impermeable walls begins to emerge, thorough monitoring continues, including the impact on fluctuation by rainfall, etc.



<Monitoring before and after closure of sea-side impermeable walls>

Installation of frozen pipes completed

Land-side impermeable walls surrounding the Unit 1-4 buildings are being installed to reduce groundwater inflow into the same.

Construction on the land side, for which freezing will start first, was completed on September 15. On the sea side, installation of frozen pipes was completed on November 9. The next step started to install pipes, etc.



Installation of frozen pipes>

Dismantling and modification of Unit 2 Reactor Building rooftop decided

To facilitate the removal of fuel assemblies and debris in the Unit 2 spent fuel pool, the scope of dismantling and modification of the existing Reactor Building rooftop was examined.

To ensure safety during the work, limit the impact on the outside of the power station and remove fuel rapidly to reduce risks, we decided to dismantle the whole rooftop above the highest floor of the Reactor Building. The dismantling will be conducted with safety prioritized above all.

Leakage from the accumulated water transfer facility, etc. into fences

Leakages were detected at the highperformance multi-nuclide removal equipment on November 2 and 25, at the Unit 2 accumulated water transfer facility on November 5, and at the desalination equipment (RO2-5) on November 15. All these leakages remained within the fences and no leakage outside the buildings was identified.

Operations resumed at the accumulated water transfer facility on November 11 after taking measures.

At the high-performance multi-nuclide removal and desalination equipment, investigations into the cause are currently underway.



* Data of Monitoring Posts (MP1-MP8.)

Data (10-minute value) of Monitoring Posts (MPs) measuring airborne radiation rate around site boundaries show 0.840 - 3.522 µSv/h (October 28 – November 24, 2015).

We improved the measurement conditions of monitoring posts 2 to 8 for precise measurement of air dose rate. Construction works such as tree-clearing, surface soil removal and shield wall setting were implemented from Feb . 10 to Apr. 18, 2012.

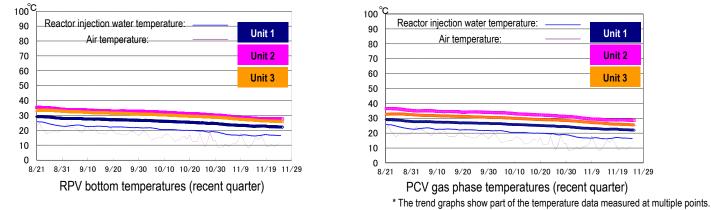
Therefore monitoring results at these points are lower than elsewhere in the power plant site.

The radiation shielding panel around monitoring post No. 6, which is one of the instruments used to measure the radiation dose of the power station site boundary, were taken off from July 10-11, 2013, since the surrounding radiation dose has largely fallen down due to further cutting down of the forests, etc

I. Confirmation of the reactor conditions

1. Temperatures inside the reactors

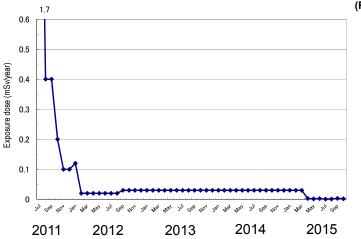
Through continuous reactor cooling by water injection, the temperatures of the Reactor Pressure Vessel (RPV) bottom and the Primary Containment Vessel (PCV) gas phase have been maintained within the range of approx. 20 to 40°C for the past month, though they vary depending on the unit and location of the thermometer.



2. Release of radioactive materials from the Reactor Buildings

As of October 2015, the density of radioactive materials newly released from Reactor Building Units 1-4 in the air and measured at the site boundaries was evaluated at approx. 6.1×10⁻¹¹ Bg/cm³ for Cs-134 and 1.4×10⁻¹⁰ Bg/cm³ for Cs-137 respectively. The radiation exposure dose due to the release of radioactive materials was less than 0.0019 mSv/year at the site boundaries.

Annual radiation dose at site boundaries by radioactive materials (cesium) released from Reactor Building Units 1-4



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Note: Different formulas and coefficients were used to evaluate the radiation dose in the facility operation plan and monthly report. The evaluation methods were integrated in September 2012. As the fuel removal from the spent fuel pool (SFP) commenced for Unit 4, the radiation exposure dose from Unit 4 was added to the items subject to evaluation since November 2013. The evaluation has been changed to a method considering the values of continuous dust monitors since FY2015, with data to be evaluated monthly and announced the following month.

3. Other indices

There was no significant change in indices, including the pressure in the PCV and the PCV radioactivity density (Xe-135) for monitoring criticality, nor was any abnormality in the cold shutdown condition or criticality sign detected.

Based on the above, it was confirmed that the comprehensive cold shutdown condition had been maintained and the reactors remained in a stabilized condition.

II. Progress status by each plan

1. Contaminated water countermeasures

To tackle the increase in accumulated water due to groundwater inflow, fundamental measures to prevent such inflow into the Reactor Buildings will be implemented, while improving the decontamination capability of water treatment and preparing facilities to control the contaminated water

- Operation of groundwater bypass \geq
- From April 9, 2014, the operation of 12 groundwater bypass pumping wells commenced sequentially to pump up and released after TEPCO and a third-party organization confirmed that its quality met operational targets.
- For pumping well Nos. 8, 9 and 12, pumping of groundwater was suspended for cleaning (No. 8: October 28 November 19; No. 9: October 6 – November 13; No. 12: from November 16).
- On November 16, when part of the control power for the groundwater bypass was suspended for preparation, all 17.
- Status of water treatment facilities including subdrains \geq
- third-party organization had confirmed that the quality of this purified groundwater met operational targets.
- Due to rising water levels of the groundwater drain pond since the closure of the sea-side impermeable walls, pumping started on November 5. As of November 25, a total of 5,744 m³ had been pumped up.
- · Regarding the Unit 1 control cable duct, at which the groundwater inflow to the Unit 1 Turbine Building was detected, (see Figure 1).

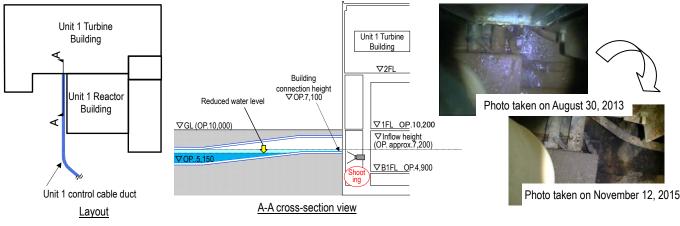


Figure 1: Unit 1 control cable duct connection - Status of groundwater inflow

- Construction status of land-side impermeable walls \geq
- To facilitate the installation of land-side impermeable walls surrounding Units 1-4 (a subsidy project of the Ministry of Economy, Trade and Industry), drilling to place frozen pipes commenced (from June 2, 2014).
- Regarding the mountain side, following the finished installation of frozen pipes on July 28, filling of brine also mountain side.
- From April 30, the freezing functioning test was underway at 18 points (58 frozen pipes, approx. 6% on the mountain side). Brine supply to the freezing functioning test points was suspended from August 21 due to the filling of brine.
- (see Figure 3). Installation of brine pipes is currently underway.

groundwater. The release started from May 21, 2014 in the presence of officials from the Intergovernmental Liaison Office for the Decommissioning and Contaminated Water Issue of the Cabinet Office. As of November 25, 2015, 148,898 m³ of groundwater had been released. The pumped-up groundwater has been temporarily stored in tanks

groundwater bypass pumps stopped due to communication failure. As no abnormality was identified in the later inspection when the relevant system had recovered, the groundwater bypass pumps were reactivated on November

To reduce the groundwater flowing into the buildings, work began to pump up groundwater from wells (subdrains) around the buildings on September 3. The pumped-up groundwater was then purified at dedicated facilities and released from September 14. As of November 25, a total of 23,928 m³ had been drained after TEPCO and a

reinvestigation of the status of groundwater inflow on November 12 identified termination of inflow. The cause of the termination was assumed to be the decreased groundwater level due to operation of subdrains, which consequently lowered the water level inside the relevant duct to below the height of the connection to the Unit 1 Turbine Building

finished on September 15. Through these works, preparation for freezing was completed for three sides on the

Regarding the sea side, drilling was completed on October 15 (for frozen pipes: 532 points, for temperature-measurement pipes: 131 points). As of November 9, installation of frozen pipes had been completed

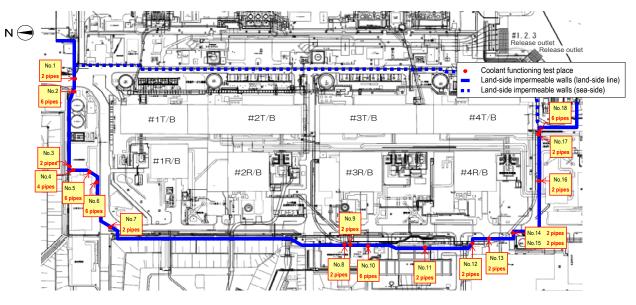


Figure 2: Freezing functioning test place on land-side impermeable walls

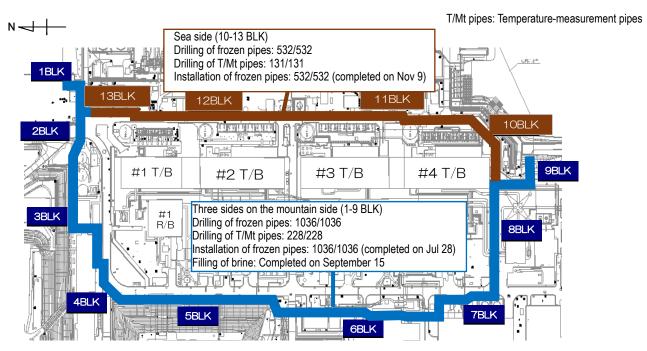


Figure 3: Drilling status for frozen-soil impermeable walls and installation of frozen pipes

- Operation of multi-nuclide removal equipment \geq
 - Regarding multi-nuclide removal equipment (existing, additional and high-performance), hot tests using radioactive water are underway (for existing equipment, System A: from March 30, 2013, System B: from June 13, 2013, System C: from September 27, 2013; for additional equipment, System A: from September 17, 2014, System B: from September 27, 2014, System C: from October 9, 2014; for high-performance equipment, from October 18, 2014).
- As of November 19, the volumes treated by existing, additional and high-performance multi-nuclide removal equipment were approx. 255,000, 230,000 and 92,000 m³ respectively (including as of November 19, approx. 9,500 m³ stored in the J1(D) tank, which contained water with a high density of radioactive materials at the System B outlet of existing multi-nuclide removal equipment).

- For Systems A and C of existing multi-nuclide removal equipment, following facility inspections and installation of inspection for System B will be conducted after operations of Systems A and C resume.
- On November 2, approx. 50 liters of leakage was detected at two points near the mesh to prevent ingress of foreign the mesh. After taking provisional countermeasures, treatment resumed from November 12.
- On November 25, a 1m x 1m of dispersal was detected on the floor due to leakage of cleaning water from an elevated vent pipe of the high-performance multi-nuclide removal equipment while cleaning with filtered water. The leakage remained within the fences and no leakage outside the fences was identified.
- 134,000 m³ had been treated.
- \geq Toward reducing the risk of contaminated water stored in tanks
- 19, approx. 147,000 m³ had been treated.
- Measures in Tank Areas \geq
- 2014 (as of November 23, 2015, a total of 39,260 m³)
- Leakage from the accumulated water transfer facility into the building
- · On November 5, the leakage detector of the accumulated water transfer facility installed in the Unit 2 Turbine 1mm outside the fences.
- · An investigation into four pipes near the leakage points revealed a crack and dimple on the surface of a pipe. After isolating the relevant pipe, the transfer resumed from November 11.
- Leakage from desalination equipment (RO2)
- On November 15, a leakage was detected from the booster pump outlet pipe joint of the desalination equipment (RO2-5) into the fences. The leakage, approx. 1m x 15m x 20mm large, remained within the fences. The cause is currently being analyzed and countermeasures considered.

additional absorption vessels to improve their performance, operations will resume from early December. An

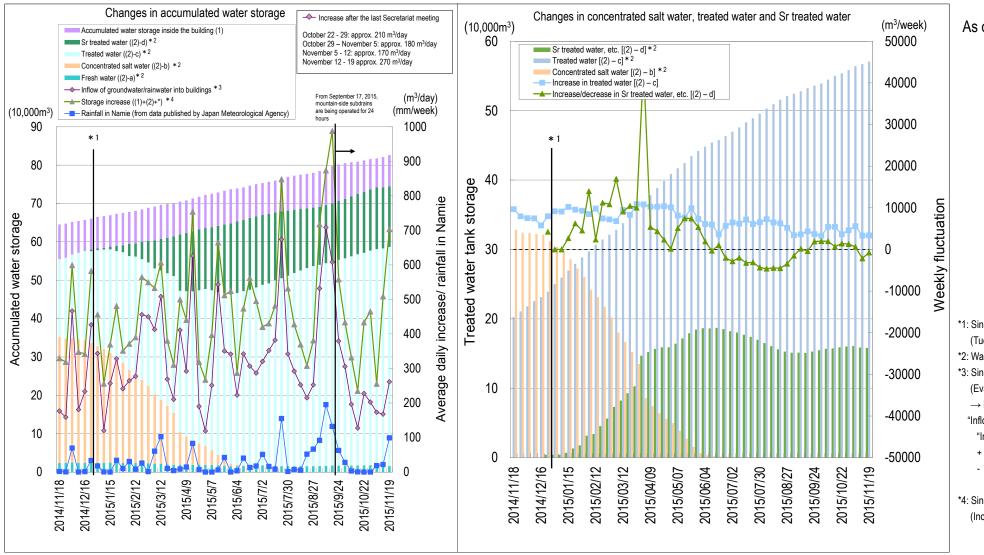
substances into the vent pipes of the high-performance multi-nuclide removal equipment filter. The cause was assumed to be the valve, which was used for the first time after being replaced, which failed to work and remained closed. This exerted pressure on a portion of the system, discharging water to vent pipes and causing it to leak from

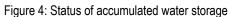
To reduce the risks of strontium-treated water, treatment by additional and high-performance multi-nuclide removal equipment is underway (additional: from May 27, high-performance: from April 15). As of November 19, approx.

Treatment measures comprising the removal of strontium by cesium absorption apparatus (KURION) (from January 6) and secondary cesium absorption apparatus (SARRY) (from December 26, 2014) are underway. As of November

Rainwater, under the release standard and having accumulated inside the fences in the contaminated water tank area, was sprinkled on site after eliminating radioactive materials using rainwater-treatment equipment since May 21,

Building issued an alert. An on-site inspection identified a drop of approx. 2 cm height inside the fences (approx. 2m x 5m x 5cm) installed at the lower part of the pipes to transfer accumulated water and a drop of approx. 5m x 5m x





2. Fuel removal from the spent fuel pools

Work to help remove spent fuel from the pool is progressing steadily while ensuring seismic capacity and safety. The removal of spent fuel from the Unit 4 pool commenced on November 18, 2013 and was completed on December 22, 2014

- Main work to help remove spent fuel at Unit 1
- On July 28, work started to remove the roof panels of the building cover. By October 5, all six roof panels had been removed. During this work, no significant change was identified in the dust densities at dust monitors and monitoring posts, etc. To facilitate the removal of steel frames which would hinder the installation of sprinklers, pre-spraying of anti-scattering agents and suction of small rubble such as concrete pieces has been underway from November 9 and 19 respectively.
- The dismantling of the building cover is being conducted with anti-scattering measures steadily implemented and safety prioritized above all.

➤ Main work to help remove spent fuel at Unit 2

- To help remove spent fuel from the pool of Unit 2 Reactor Building, dismantling of hindrance buildings around the Reactor Building has been underway since September 7 to clear a work area to install large heavy-duty machines, etc.
- Scope of dismantling and modification of Unit 2 Reactor Building rooftop \geq
- To facilitate the removal of fuel assemblies and debris in the spent fuel pool, the scope of dismantling and

modification of the existing Reactor Building rooftop was examined. To ensure safety during the work, limit the impacts on the outside of the power station and removing fuel rapidly to reduce risks, we decided to dismantle the whole rooftop of the operating floor. The dismantling will be conducted with anti-scattering measures for dust steadily implemented based on the experience in Units 3 and 4 and with safety prioritized above all. In tandem, there is also consideration as to whether or not the same structure will be used to remove pool fuel assemblies and debris, which will be finally decided approx. two years later.

- Main work to help remove spent fuel at Unit 3
- Removal of rubble inside the spent fuel pool using large cranes was concluded on November 21.

3. Fuel debris removal

In addition to decontamination and shield installation to improve PCV accessibility, technology was developed and data gathered as required to prepare to remove fuel debris (such as investigating and repairing PCV leak locations)

- Investigation into the Main Stream Valve Room on Unit 1 Reactor Building 1st floor
- investigated in December.
- Progress of decontamination around Unit 2 X-6 penetration \geq
- To facilitate the investigation into the status of the platform inside the Unit 2 PCV pedestal (A2 investigation),

As of November 26, 2015

*1: Since January 1, 2015, the data collection days have been changed (Tuesdays \rightarrow Thursdays) *2: Water amount with which water-level gauge indicates 0% or more *3: Since September 10, 2015, the data collection method has been changed (Evaluation based on increased in storage: in buildings and tanks → Evaluation based on increase/decrease in storage in buildings) "Inflow of groundwater/rainwater into buildings" =

"Increase/decrease of water held in buildings" + "Transfer from buildings to tanks"

- "Transfer into buildings (water injection into reactors and transfer from well points, etc.)

*4: Since April 23, 2015, the data collection method has been changed

(Increase in storage $((1)+(2) \rightarrow (1)+(2)+^*)$)

To confirm the need for a dose reduction that may facilitate future investigations inside PCV and repair, an investigation of the Main Steam Valve Room has been underway from November 18. The Airlock Room will also be

decontamination is underway around X-6 penetration from which the investigation device will be inserted (removal of eluted materials: October 30 - November 5, decontamination by steam: November 11-13, chemical decontamination: from November 17 to early December). If the radiation dose is not reduced by chemical decontamination, the floor surface will also be ground.

Investigation into the Unit 3 PCV equipment hatch

• In 2011, high-dose puddles were identified in and around the grooves of the shield-plug transfer rail of the Unit 3 PCV equipment hatch. Due to potential leakage from the equipment hatch seal, the status of this seal, etc. was investigated using a small camera on September 9, 2015. This will be followed by further investigation on November 26 and 27 using a small self-traveling investigation device to access the equipment hatch more closely and confirm the status of the seal, etc.



Figure 5: Decontamination around Unit 2 X-6 penetration

4. Plans to store, process and dispose of solid waste and decommission of reactor facilities

Promoting efforts to reduce and store waste generated appropriately and R&D to facilitate adequate and safe storage, processing and disposal of radioactive waste

Figure 6: Small investigation device for Unit 3 PCV

equipment hatch

Management status of rubble and trimmed trees

- As of the end of October, the total storage volume of concrete and metal rubble was approx. 165,400 m³ (+4,400 m³ compared to at the end of September, with an area-occupation rate of 62%). The total storage volume of trimmed trees was approx. 84,200 m³ (+2,100 m³ compared to at the end of September, with an area-occupation rate of 79%). The increase in rubble was mainly attributable to construction related to facing and the installation of tanks. The increase in trimmed trees was mainly attributable to construction related to facing and the acceptance of branches and leaves accumulated in temporary storage.
- Management status of secondary waste from water treatment
- As of November 19, 2015, the total storage volume of waste sludge was 597 m³ (area-occupation rate: 85%) and that of concentrated waste fluid was 9,315 m³ (area-occupation rate: 47%). The total number of stored spent vessels, High-Integrity Containers (HICs) for multi-nuclide removal equipment, etc. was 2,877 (area-occupation rate: 48%).
- Test operation of Radioactive Waste Incinerator started \geq
- From November 25, a cold test incinerating dummy waste, which generates no contamination, started to verify facility-wide functions and performance. Following the cold test, which will continue until the end of December, pre-operation and hot tests using actual contaminated waste will be conducted to start operations within this fiscal year.

5. Reactor cooling

The cold shutdown condition will be maintained by cooling the reactor by water injection and measures to complement the status monitoring will continue

- Investigation inside Unit 3 PCV and the installation of a permanent monitor
- Thermometers and a water-level gage will be installed from the Unit 3 PCV penetration (X-53) into the PCV (scheduled for December 10-17).

6. Reduction in radiation dose and mitigation of contamination

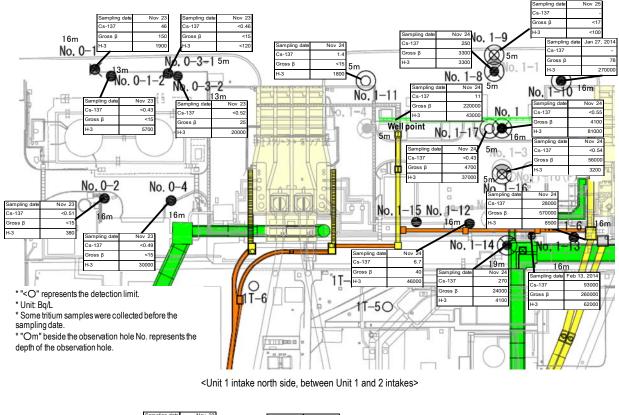
Effective dose-reduction at site boundaries and purification of port water to mitigate the impact of radiation on the external environment

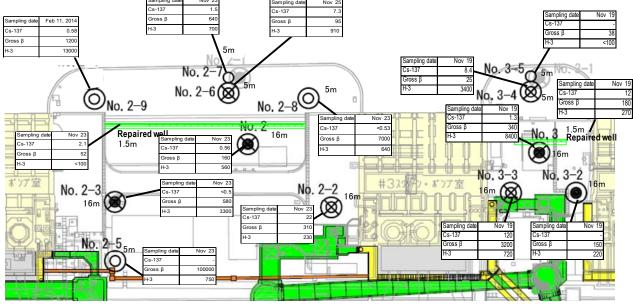
- Status of groundwater and seawater on the east side of Turbine Building Units 1 to 4
- Bq/L.
- resumed.
- Regarding radioactive materials in the groundwater near the bank between the Unit 2 and 3 intakes, though the tritium density at groundwater Observation Hole No. 2-3 has remained constant at around 1,000 Bg/L, it has been increasing from September and currently stands at around 3,000 Bg/L. Though the density of gross β radioactive materials at groundwater Observation Hole No. 2-5 has remained constant at around 10,000 Bg/L, it has been increasing since November and currently stands at around 100,000 Bg/L. Sampling frequency increased from monthly to weekly. Since December 18, 2013, pumping of groundwater continued at the well point between the Unit 2 and 3 intakes and since October 14, 2015, it was shifted to pumping at the repaired well point.
- intakes. Since September 17, 2015, it was shifted to pumping at the repaired well point.
- Regarding the radioactive materials in seawater outside the sea-side impermeable walls and within the open channels of Units 1-4, as well as those inside the port, the density was decreasing due to the effect of the completed installation and the connection of steel pipe sheet piles for the sea-side impermeable walls.
- Regarding the radioactive materials in seawater outside the port, the densities of cesium 137 and tritium have remained within the same range previously recorded.
- Regarding the Unit 1 drainage channel, though the density of radioactive materials increased in September, it has subsequently been decreasing. Cleaning by the mobile treatment device will start after completing the preparation.
- In response to multiple overflow of waste water from temporary fences of the K drainage channel during heavy rainfall, provisional countermeasures are underway pending the construction of a new drainage channel. These include transferring waste water through pumps from the branch channel in the groundwater bypass area in the upper stream of K drainage to the relay pit of B drainage channel (from November 2).

Regarding the radioactive materials in groundwater near the bank on the north side of the Unit 1 intake, the tritium density has remained constant at around 10,000 Bg/L at groundwater Observation Hole No. 0-3-2. However, after decreasing from September, the density has been increasing since October and currently stands at around 20,000

Regarding the groundwater near the bank between the Unit 1 and 2 intakes, though the tritium density at groundwater Observation Hole No. 1-11 has remained constant at around 10,000 Bg/L, it has been decreasing since September and currently stands at around 2,000 Bg/L. The density of gross ß radioactive materials at groundwater Observation Hole No. 1 has been increasing and currently stands at around 10,000 Bg/L since February 2015. Though the density of gross β radioactive materials at groundwater Observation Hole No. 1-16 has remained constant at around 200,000 Bg/L, after decreasing from September, it has been increasing since October and currently stands at around 100,000 Bg/L. Water pumping at the repaired well point started (from October 14). Since August 15, 2013, pumping of groundwater continued at the well point between the Unit 1 and 2 intakes. Since October 14, 2015, it was shifted to pumping at the repaired well point. Since October 24, pumping at the well point

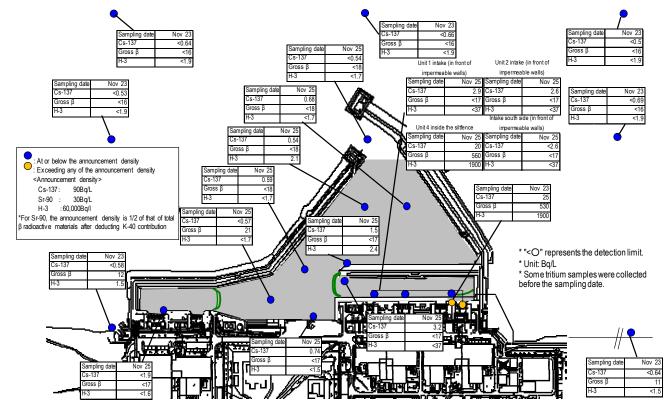
Regarding radioactive materials in the groundwater near the bank between the Unit 3 and 4 intakes, the tritium density at groundwater Observation Hole No. 3-4 has been increasing since August and currently stands at around 3,000 Bg/L. Since April 1, 2015, pumping of groundwater continued at the well point between the Unit 3 and 4





<Between Unit 2 and 3 intakes, between Unit 3 and 4 intakes>

Figure 7: Groundwater density on the Turbine Building east side



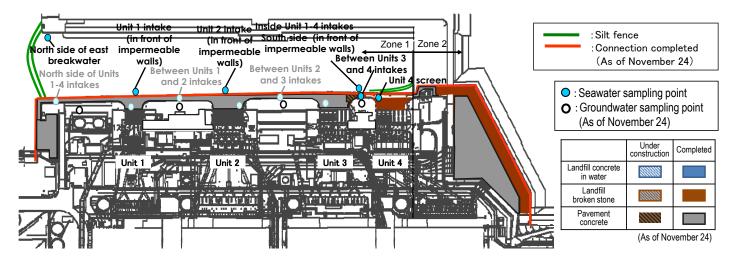


Figure 9: Progress status of impermeable walls on the sea side

7. Review of the number of staff required and efforts to improve the labor environment and conditions

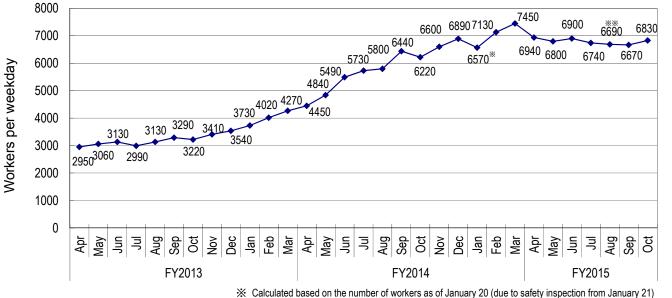
Securing appropriate staff long-term while thoroughly implementing workers' exposure dose control. Improving the work environment and labor conditions continuously based on an understanding of workers' on-site needs

- Staff management
 - The monthly average total of people registered for at least one day per month to work on site during the past work on site.

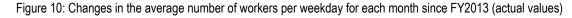
Figure 8: Seawater density around the port

quarter from July to September 2015 was approx. 13,800 (TEPCO and partner company workers), which exceeded the monthly average number of actual workers (approx. 10,800). Accordingly, sufficient people are registered to

- · It was confirmed with the prime contractors that the estimated manpower necessary for the work in December (approx. 6,600 per day: TEPCO and partner company workers)* would be secured at present. The average numbers of workers per day for each month (actual values) were maintained, with approx. 3,000 to 7,500 since FY2014 (see Figure 10). * Some works for which contractual procedures have yet to be completed are excluded from the December estimate
- The number of workers from Fukushima Prefecture has remained the same but the number from outside the prefecture has increased slightly. Accordingly, the local employment ratio (TEPCO and partner company workers) as of October remained at around 50% with a slight decline.
- The average exposure dose of workers remained at approx. 1 mSv/month during FY2013, FY2014 and FY2015. (Reference: Annual average exposure dose 20 mSv/year = 1.7 mSv/month).
- · For most workers, the exposure dose was sufficiently within the limit and allowed them to continue engaging in radiation work.



XXX Calculated based on the number of workers from August 3-7, 24-28 and 31 (due to overhaul of heavy machines)



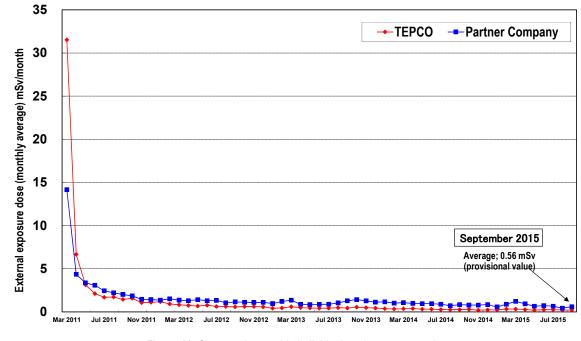


Figure 11: Changes in monthly individual worker exposure dose (monthly average exposure dose since March 2011)

- Results of questionnaire survey for workers to improve the labor environment \geq 86.4% (6,527 workers) of the personnel responded to the guestionnaire. The results showed that compared with the last survey, evaluation of the labor environment had improved. Approx. 80% of respondents evaluated the following items achieved this year as "good" or "reasonably good": operation start of the large rest house and dining room and expansion of the area where wearing full-face masks is not required. Based on feedback from workers, further improvements will be made, such as setting up parking and rest houses on and off-site and installing shower facilities.
- Measures to prevent infection and expansion of influenza and norovirus \geq
- entry/exit and mandatory wearing of masks in working spaces).
- Status of influenza and norovirus cases \geq
- Until the 47th week of 2015 (November 16-22, 2015), there was one case of influenza infections and one case of norovirus infections. The totals for the same period for the previous season showed four cases of influenza infections and no case of norovirus infections. The totals for the entire previous season (November 2014 to March 2015) showed 353 cases of influenza infections and 10 cases of norovirus infections.

8. Other

- Smoke from the power panel of Main Anti-Earthquake Building \geq
- Building 1st floor and consequently generating smoke.
- Opening of the "1 FOR ALL JAPAN" website \geq
- Aiming to provide workers with information on the "workplace", convey messages from their fellows and supporters website (http://1f-all.jp/) was opened in October for workers on site and their families.

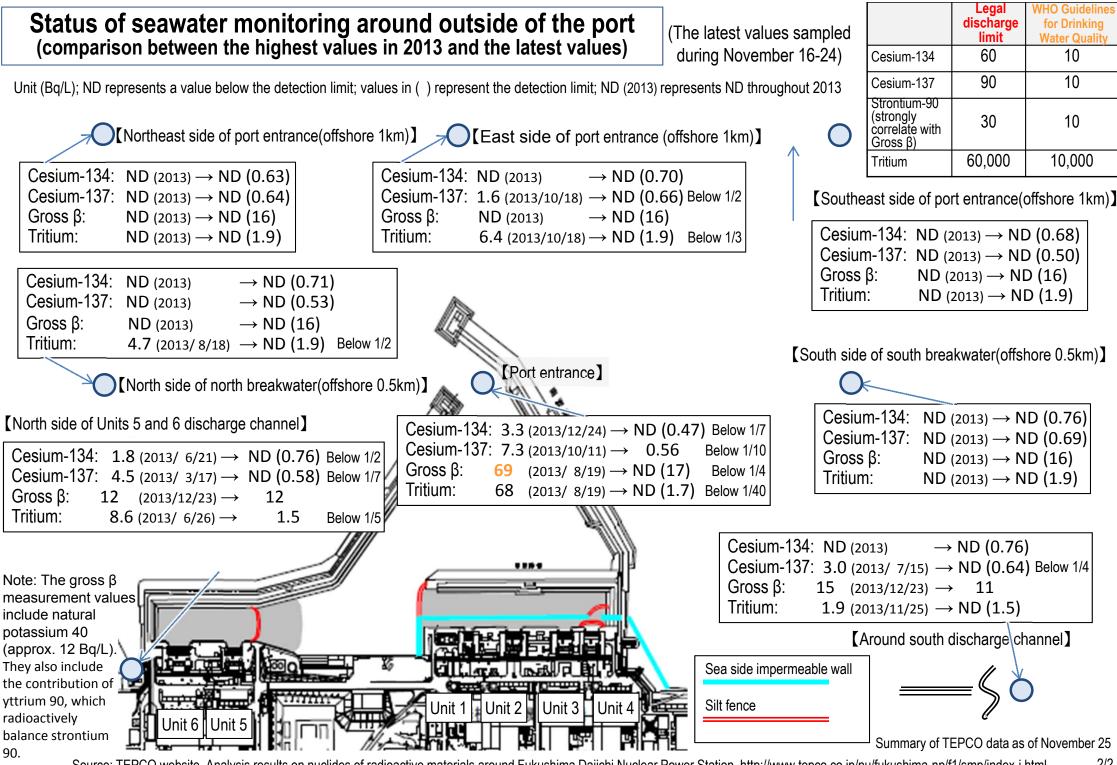
Since October, measures for influenza and norovirus have been implemented, including free influenza vaccinations (subsidized by TEPCO) in the Fukushima Daiichi Nuclear Power Station (from October 28 to December 4) and medical clinics around the site (from November 2, 2015 to January 29, 2016) for partner company workers. As of November 20, a total of 5,452 workers had been vaccinated. In addition, a comprehensive range of other measures is also being implemented, including daily actions to prevent infection and expansion (measuring body temperature, health checks and monitoring infection status) and response after detecting possible infections (control of swift

On November 19, smoke was identified from the grounding current-limiting resistor in the power room on the Main Anti-Earthquake Building 1st floor (but soon stopped). The cause was assumed to be an iron pin used to fix the ropes for dividing the area near the slope on the west side of the old administration office building on site, which inadvertently came into contact with a high-pressure cable of the common metal-clad (M/C) 1A system on site, generating a current in the grounding current-limiting resistor in the power room on the Main Anti-Earthquake

and help workers engage in long-term decommissioning safely and maintain motivation, the "1 FOR ALL JAPAN"

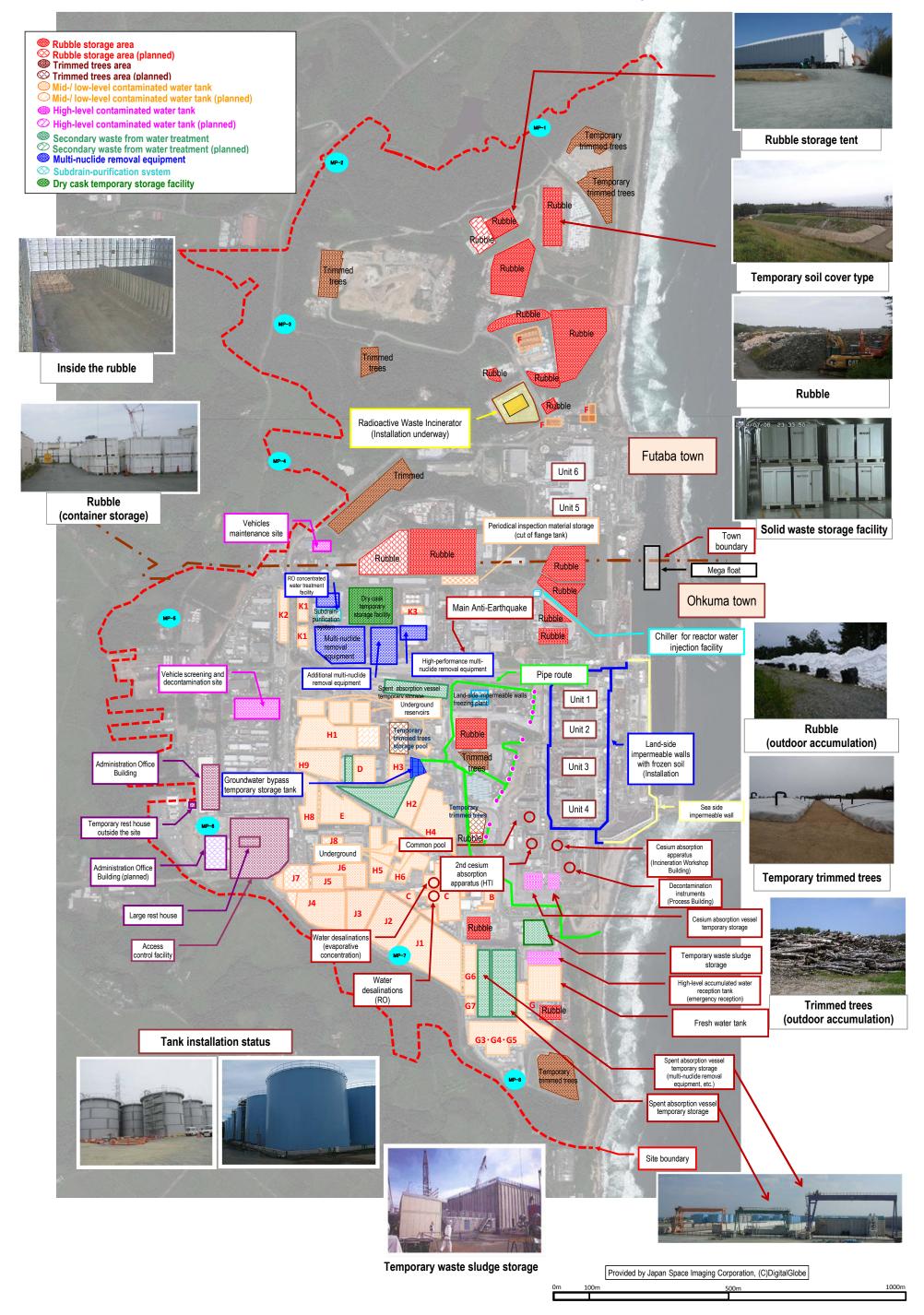
Appendix 1

Status of seawater monitoring within the port (comparison between the highest values in 2013 and the latest values) "The highest value" \rightarrow "the latest value (sampled during November 16-24)"; unit (Bg/L); ND represents a value below the detection limit Sea side impermeable wall Source: TEPCO website Analysis results on nuclides of radioactive materials around Fukushima Daiichi Nuclear Cesium-134: 3.3 $(2013/10/17) \rightarrow ND(0.35)$ Below 1/9 Power Station http://www.tepco.co.jp/nu/fukushima-np/f1/smp/index-j.html Silt fence Cesium-137: 9.0 (2013/10/17) \rightarrow 0.51 Below 1/10 Cesium-134: 0.73 Gross β: 74 $(2013/8/19) \rightarrow ND(17)$ Below 1/4 Cesium-134: 3.3 (2013/12/24) → ND(0.47) Below 1/7 Cesium-137: 3.7 Tritium: 67 (2013/ 8/19) → 2.1 Below 1/30 Cesium-137: 7.3 (2013/10/11) \rightarrow 0.56 Below 1/10 Gross β : ND(17) Gross β : **69** $(2013/8/19) \rightarrow ND(17)$ Below 1/4 Tritium: 2.4 Cesium-134: 4.4 (2013/12/24) → ND(0.59) Below 1/7 Tritium: 68 $(2013/8/19) \rightarrow ND(1.7)$ Below 1/40 Cesium-137: 10 (2013/12/24) → 0.96 Below 1/10 Gross β: Cesium-134: 3.5 (2013/10/17) \rightarrow ND(0.50) Below 1/7 60 $(2013/7/4) \rightarrow ND(17)$ Below 1/3 Port entrance Cesium-137: 7.8 (2013/10/17) → Tritium: 59 $(2013/8/19) \rightarrow ND(1.7)$ Below 1/30 1.1 Below 1/7 Gross β: **79** $(2013/8/19) \rightarrow ND(17)$ Below 1/4 Cesium-134: 5.0 (2013/12/2) → ND(0.50)Below 1/10 Tritium: 60 $(2013/8/19) \rightarrow ND(1.7)$ Below 1/30 Cesium-137: 8.4 (2013/12/2) → 0.52 Below 1/10 Cesium-134: 32 (2013/10/11) → ND(2.4) Below 1/10 Gross β: 69 ND(17) Below 1/4 $(2013/8/19) \rightarrow$ South side Cesium-137: 73 (2013/10/11) → Below 1/20 3.5 Tritium: Below 1/30 52 ND(1.7) (2013/8/19) → in the port Gross β: 320 (2013/ 8/12) → ND(15) Below 1/20 Cesium-134: 2.8 $(2013/12/2) \rightarrow ND(1.6)$ Tritium: Below 6/10 $510(2013/9/2) \rightarrow ND(37)$ Below 1/10 Cesium-137: 5.8 $(2013/12/2) \rightarrow ND(2.2)$ Below 1/2 [East side in the port] Cesium-134: ND(1.9) ND(2.5) Cesium-134: Gross β: 46 $(2013/8/19) \rightarrow ND(17)$ Below 1/2 Cesium-137: Cesium-137: 5.3 7.4 Gross β: Gross B: Tritium: ND(17) 18 $(2013/8/19) \rightarrow ND(1.6)$ 24 Below 1/10 [Port center] ND(37) * ND(37) * Tritium: Tritium: WHO West side Legal Cesium-134: ND(1.9) Guidelines for discharge in the port] Cesium-137: 5.8 Drinking limit Gross B: ND(15) Water Quality Tritium: ND(37) * 60 10 Cesium-134 [North side in the port] * Monitoring commenced in or after 10 In front of shallow 90 Cesium-137 March 2014 In front of Unit 6 intake draft quay Strontium-90 (strongly 30 10 Cesium-134: **62** (2013/ 9/16) → 3.5 Below 1/10 correlate with Cesium-137: 140 (2013/ 9/16)→ 21 Below 1/6 Gross β) 60.000 10.000 Gross β: 580 360 (2013/ 8/12)→ Tritium Tritium: 400 (2013/ 8/12)→ 1,900 Cesium-134: 5.3 (2013/8/ 5) \rightarrow ND(0.51) Below 1/10 Cesium-134: 28 (2013/ 9/16)→ 5.1 Below 1/5 Note: The gross β measurement Cesium-137: 8.6 (2013/8/ 5) → 0.71 Below 1/10 Cesium-137: 53 (2013/12/16)→ 21 values include natural potassium 40 Below 1/2 (approx. 12 Bg/L). They also include Summary of Gross β: 40 20 Below 1/2 $(2013/7/3) \rightarrow$ 560 Gross β: **390** (2013/ 8/12)→ the contribution of yttrium 90, which TEPCO data as Tritium: 340 $(2013/6/26) \rightarrow ND(1.5)$ Below 1/200 Tritium: 650 (2013/ 8/12)→ 1,900 radioactively balance strontium 90. of November 25



Source: TEPCO website, Analysis results on nuclides of radioactive materials around Fukushima Daiichi Nuclear Power Station, http://www.tepco.co.jp/nu/fukushima-np/f1/smp/index-j.html

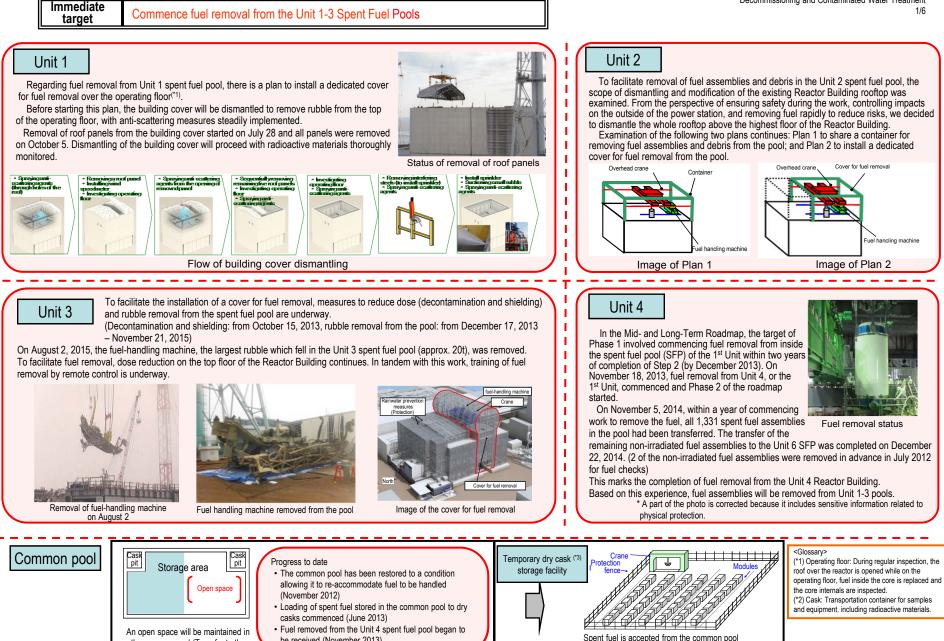
TEPCO Fukushima Daiichi Nuclear Power Station Site Layout



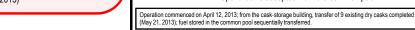
Reference

Progress toward decommissioning: Fuel removal from the spent fuel pool (SFP)

November 26, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 1/6



An open space will be maintained in the common pool (Transfer to the temporary dry cask storage facility)



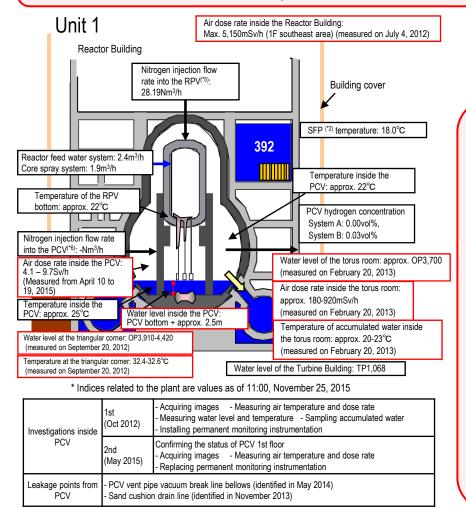
Immediate target

Identify the plant status and commence R&D and decontamination toward fuel debris removal

Val November 26, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 2/6

Investigation into TIP Room of the Unit 1 Reactor Building

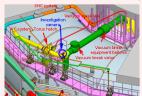
- To improve the environment for future investigations inside the PCV, etc., an investigation was conducted from September 24 to October 2 at the TIP Room(*1). (Due to high dose around the entrance in to the TIP Room, the investigation of dose rate and contamination distribution was conducted through a hole drilled from the walkway of the Turbine Building, where the dose was low)
- The investigative results identified high dose at X-31 to 33 penetrations(*2) (instrumentation penetration) and low dose at other parts.
- As it was confirmed that work inside the TIP room would be available, the next step will include identification of obstacles which will interfere the work inside the TIP Room and formulation of a plan for dose reduction.



Investigation in the leak point detected in the upper part of the Unit 1 Suppression Chamber $(S/C^{(*3)})$

Investigation in the leak point detected in the upper part of Unit 1 S/C from May 27, 2014 from one expansion joint cover among the lines installed there. As no leakage was identified from other parts, specific methods will be examined to halt the flow of water and repair the PCV.





Leak point

Image of the S/C upper part investigation

Status of equipment development toward investigating inside the PCV

Prior to removing fuel debris, to check the conditions inside the Primary Containment Vessel (PCV), including the location of the fuel debris, investigation inside the PCV is scheduled.

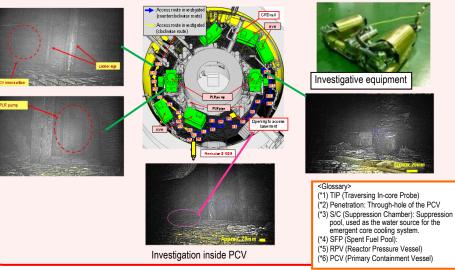
[Investigative outline]

Inserting equipment from Unit 1 X-100B penetration(¹⁵⁾ to investigate in clockwise and counter-clockwise directions.

[Status of investigation equipment development]

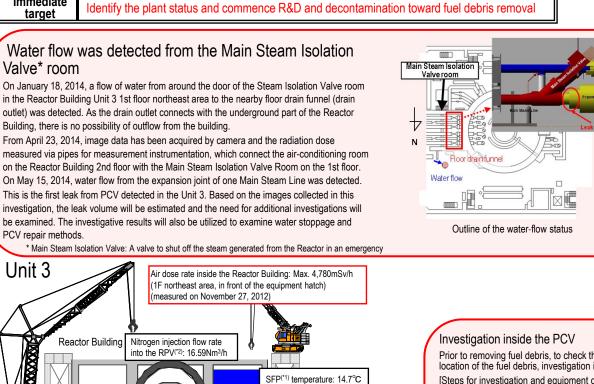
Using the crawler-type equipment with a shape-changing structure which allows it to enter the PCV from the narrow access entrance (bore: 100mm) and stably move on the grating, a field demonstration was implemented from April 10 to 20, 2015.

Through this investigation, information including images inside the PCV 1st floor and airborne radiation was obtained. The
investigation also confirmed the absence of obstacles around the access aperture leading to the basement floor, which will be used in
the next investigation. These results will be considered in next investigation of the PCV basement floor.



Progress toward decommissioning: Works to identify the plant status and toward fuel debris removal

November 26 2015 Secretariat of the Team for Countermeasures for Immediate Identify the plant status and commence R&D and decontamination toward fuel debris removal Decommissioning and Contaminated Water Treatment target 3/6 enepaton Penepaton Penepaton Penepaton Penetration (1) Installation of an RPV thermometer and permanent PCV supervisory instrumentation Investigative results on torus room walls (CUN-17) W-20) (MSC-14) (RCW-29) (FRC-41) The torus room walls were investigated (on the north side (1) Replacement of the RPV thermometer of the east-side walls) using equipment specially developed • As the thermometer installed at the Unit 2 RPV bottom after the earthquake had broken in February 2014, it was excluded Φ Ð for that purpose (a swimming robot and a floor traveling from the monitoring thermometers. n robot). · On April 2014, removal of the broken thermometer failed and was suspended. Rust-stripping chemicals were injected and the broken thermometer was removed on January 2015. A new thermometer was reinstalled on March. The thermometer At the east-side wall pipe penetrations (five points), "the South side North side has been used as a part of permanent supervisory instrumentation since April. status" and "existence of flow" were checked Penetrations investigated (2) Reinstallation of the PCV thermometer and water-level gauge (Investigative equipment · A demonstration using the above two types of underwater R/B 1st floor Some of the permanent supervisory instrumentation for PCV could not be installed in the planned locations due to Swimming robot insert point) wall investigative equipment showed how the equipment T/B Fast interference with existing grating (Áugust 2013). The instrumentation was removed on May 2014 and new instruments R/B torus room could check the status of penetration. -side were reinstalled on June 2014. The trend of added instrumentation will be monitored for approx, one month to evaluate its wall Swimming robot Regarding Penetrations 1 - 5, the results of checking the validity. Tracer The measurement during the installation confirmed that the water level inside the PCV was approx. 300mm from the sprayed tracer (*5) by camera showed no flow around the S/C bottom penetrations. (investigation by the swimming robot) Sona Floor traveling Unit 2 Regarding Penetration 3, a sonar check showed no flow Floor traveling around the penetrations. (investigation by the floor traveling Air dose rate inside the Reactor Building: Max. 4.400mSv/h (1F southeast area. robot upper penetration^(*1) surface) (measured on November 16, 2011) robot) Image of the torus room east-side cross-sectional investigation Reactor Building Nitrogen injection flow rate into the RPV(*3). Status of equipment development toward investigating inside the PCV 15.60Nm3/h Prior to removing fuel debris, to check the conditions inside the Primary Containment Vessel (PCV), including SFP(*2) temperature: 21.2°C the location of the fuel debris, investigations inside the PCV are scheduled. [Investigative outline] 615 Inserting the equipment from Unit 2 X-6 penetration⁽¹⁾ and accessing inside the pedestal using the CRD rail Reactor feed water system: 1.9m3/h to conduct investigation. Core spray system: 2.3m3/h [Status of investigative equipment development] Based on issues confirmed by the CRD rail status investigation conducted in August 2013, the investigation Temperature inside the PCV: method and equipment design are currently being examined. Temperature of the RPV approx. 28°C • As a portion of shielding blocks installed in front of X-6 penetration could not be moved, a removal method bottom: approx. 27°C using small heavy machines was planned. The work for removing these blocks resumed on September 28 PCV hydrogen concentration and removal of interfering blocks for future investigations was also completed on October 1. System A: 0.04vol% Nitrogen injection flow rate System B: 0.04vol% into the PCV(*4): -Nm3/h Water level of the torus room: approx. OP3.270 (measured on June 6, 2012) Errort camera & light Air dose rate inside Alternative shield Self-traveling equipment Pan & tilt function the PCV: Max. approx. Air dose rate inside the torus room (draft plan) 73Sv/h Isolation valve 30-118mSv/h(measured on April 18, 2012) Isolation valve 6-134mSv/h(measured on April 11, 2013) Insertion too Temperature inside the Chamber X-6 penetratio PCV: approx. 30°C Water level at the triangular corner: OP3,050-3,190 7. Avoiding rail holding tool (measured on June 28, 2012) Water level inside the PCV: 777 Temperature at the triangular corner: 30.2-32.1°C 37 PCV bottom + approx. 300mm Issues before using X-6 penetration (measured on June 28, 2012) 5. Avoiding the 1. Removal of existing shield in front of the Measurement equipment foothold Water level of the Turbine Building: TP1,586 penetration 6. Crossing over **Turbine Building** Installation of alternative shield deposit on the 8. Crossing over the * Indices related to plant are values as of 11:00. November 25, 2015 3. Boring in the penetration hatch space between rai 4. Removal of inclusion of the penetration and niatform 9. Travel on the grating 1st (Jan 2012) Acquiring images - Measuring air temperature This plan may be changed depending on the future examination status Investigative issues inside the PCV and equipment configuration (draft plan) Investigations 2nd (Mar 2012) Confirming water surface - Measuring water temperature - Measuring dose rate inside PCV 3rd (Feb 2013 - Jun 2014) <Glossarv> Acquiring images - Sampling accumulated water (*1) Penetration: Through-hole of the PCV (*2) SFP (Spent Fuel Pool) Measuring water level - Installing permanent monitoring instrumentation (*3) RPV (Reactor Pressure Vessel) (*4) PCV (Primary Containment Vessel) Leakage points No leakage from torus room rooftop (*5) Tracer: Material used to trace the fluid flow. Clay particles No leakage from all inside/outside surfaces of S/C from PC



Temperature inside the PCV:

PCV hydrogen concentration

System A: 0.06vol%

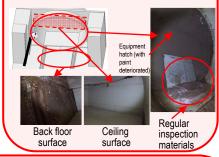
System B: 0.06vol%

approx. 25°C

November 26 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 4/6

Investigative results into PCV equipment hatch

- High-dose puddles were identified around PCV equipment hatch in the past. Due to the possibility of leakage from the equipment hatch seal, an investigation using a small camera was conducted on September 9
- The investigation identified no leakage from nor distortion of the equipment hatch, while detecting leakage from the ceiling and a deposit of coating films on the floor.



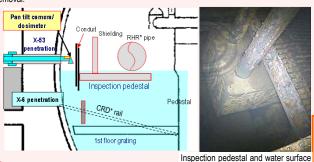
Prior to removing fuel debris, to check the conditions inside the Primary Containment Vessel (PCV) including the location of the fuel debris, investigation inside the PCV was conducted

[Steps for investigation and equipment development]

Investigation from X-53 penetration(*4)

- From October 22-24, the status of X-53 penetration, which may be under the water and which is scheduled for use to investigate the inside of the PCV, was investigated using remote-controlled ultrasonic test equipment. Results showed that the penetration is not under the water
- For the purpose of confirming the status inside the PCV, an investigation device was inserted into the PCV from X-53 penetration on October 20 and 22, 2015 to obtain images, data of dose and temperature and sample accumulated water. No damage was identified on the structure and walls inside the PCV and the water level was almost identical with the estimated value. In addition, the dose inside the PCV was confirmed to be lower than in other Units.

 In the next step, the obtained information will be analyzed to be utilized in the consideration about the policy for future fuel debris removal



<Glossarv> (*1) SFP (Spent Fuel Pool) (*2) RPV (Reactor Pressure Vessel) (*3) PCV (Primary Containment Vessel) (*4) Penetration: Through-hole of the PCV

Water level of the torus room: approx. OP3,370 Max. approx. 1Sv/h (measured on June 6, 2012) (measured on October 20, 2015) Air dose rate inside the torus room: 100-360mSv/h Water level inside the PCV: approx. (measured on July 11, 2012) OP11.800 (measured on October 20, 2015) Water level of the Turbine Building: TP1.447 Water level at the triangular corner: OP3,150 (measured on June 6, 2012)

* Indices related to plant are values as of 11:00. November 25, 2015

Immediate

Reactor feed water system: 2.0m3/h

Temperature of the RPV

bottom: approx. 25°C

Nitrogen injection flow rate

into the PCV(*3): -Nm3/h

Air dose rate inside the PCV:

Core sprav system: 2.4m3/h

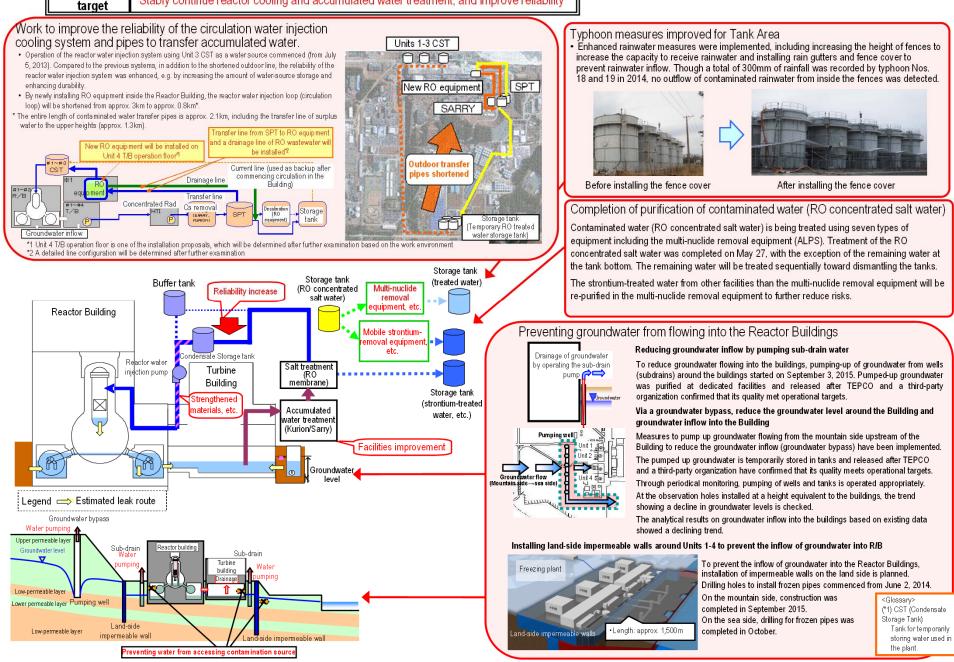
Investigations inside PCV	1st (Oct – Dec 2015)	Acquiring images - Measuring air temperature and dose rate Measuring water level and temperature - Sampling accumulated water Installing permanent monitoring instrumentation (scheduled for December 2015)
Leakage points from PC	- Main steam pipe bellows (identified in May 2014)	

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Progress toward decommissioning: Work related to circulation cooling and accumulated water treatment line

November 26, 2015 Secretariat of the Team for Countermeasures for Decommissioning and Contaminated Water Treatment 5/6

Immediate Stably continue reactor cooling and accumulated water treatment, and improve reliability



November 26, 2015 Secretariat of the Team for Countermeasures for Decommissioning

Progress toward decommissioning: Work to improve the environment within the site

and Contaminated Water Treatment

6/6

Immediate targets - Reduce the effect of additional release from the entire power station and radiation from radioactive waste (secondary water treatment waste, rubble, etc.) generated after the accident, to limit the effective radiation dose to below 1mSv/year at the site boundaries. - Prevent contamination expansion in sea, decontamination within the site

