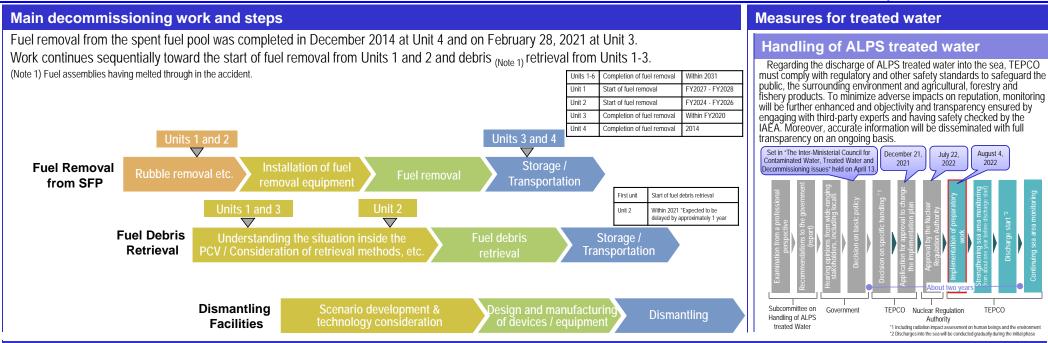
Outline of Decommissioning, Contaminated Water and Treated Water Management Secretariat of the Team for Countermeasures for Decommissioning, Contaminated Water and Treated Water Management



Contaminated water management - triple-pronged efforts -

(1) Efforts to promote contaminated water management based on the three basic policies (1) "Remove" the source of water contamination (2) "Redirect" fresh water from contaminated areas 3 "Retain" contaminated water from leakage

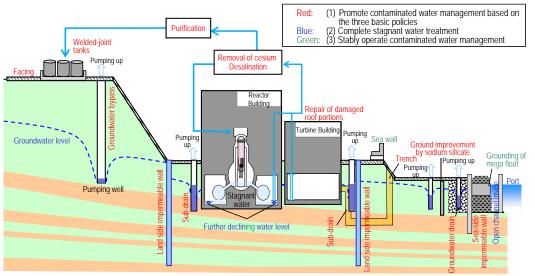
- Strontium-reduced water from other equipment is being re-treated in the Advanced Liquid Processing System (ALPS: multi-nuclide removal equipment) and stored in welded-joint tanks.
- Multi-layered contaminated water management measures, including land-side impermeable walls and sub-drains, have stabilized the groundwater at a low level and the increased contaminated water generated during rainfall is being suppressed by repairing damaged portions of building roofs, facing onsite, etc. Through these measures, the generation of contaminated water was reduced from approx. 540 m³/day (in May 2014) to approx. 130 m³/day (in FY2021).
- Measures continue to further suppress the generation of contaminated water to 100 m³/day or less within 2025.

(2) Efforts to complete stagnant water treatment

- To reduce the stagnant water levels in buildings as planned, work to install additional stagnant water transfer equipment is underway. At present, the floor surface exposure condition can be maintained except for the Unit 1-3 Reactor Buildings, Process Main Building and the High Temperature Incinerator Building.
- In 2020, treatment of stagnant water in buildings was completed, except for the Unit 1-3 Reactor Buildings, Process Main Building and High-Temperature Incinerator Building. For Reactor Buildings, the amount of stagnant water there will be reduced to about half the amount at the end of 2020 during the period FY2022-2024.
- For zeolite sandbags on the basement floors of the Process Main Building and High-Temperature Incinerator Building, measures to reduce the radiation dose are being examined with stabilization in mind.

(3) Efforts to stably operate contaminated water management

 Various measures are underway to prepare for tsunamis. For heavy rain, sandbags are being installed to suppress direct inflow into buildings while work to close openings in buildings and install sea walks to enhance drainage channels and other measures is being implemented as planned.



Progress Status and Future Challenges of the Mid-and-Long-Term Roadmap toward Decommissioning of TEPCO Holdings Fukushima Daiichi Nuclear Power Station (Outline)

Progress status

The temperatures of the Reactor and the Primary Containment Vessel of Units 1-3 have been maintained stable. There was no significant change in the concentration of radioactive materials newly released from Reactor Buildings into the air. It was concluded that the comprehensive cold shutdown condition had been maintained.

Progress of work inside and outside the site toward installing the Unit 1 large cover

Outside the site, the ground assembly of steel frames and others has been underway as part of efforts to install a large cover, which was completed for the temporary gantry and lower structure and approx. 40% for the upper structure.

Within the site, anchors and base plates are being installed to support the large cover. A temporary gantry is also being installed from the part where anchors and base plates were installed.

As preparation to mitigate any dust scattering during work, as well as sprinkling water using crane, equipment to spray water over the Reactor Building operating floor was installed on the rooftop of the Unit 1 Turbine Building as part of enhanced measures.

After installing a large cover over the Reactor Building in around FY2023, the rubble inside the cover will be removed.



<Work outside the site (August 8, 2022)>

Progress of work around the Unit 1/2 Radioactive Waste Treatment Building

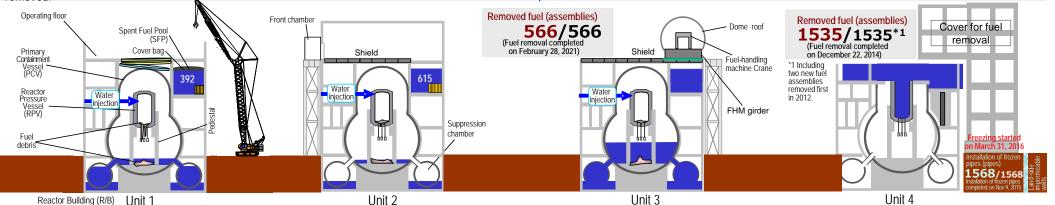
To reduce the risk of delaying the process to remove the pipes of the Unit 1/2 Standby Gas Treatment System (SGTS) and install the Unit 1 cover, the process of work around the Unit 1/2 Radioactive Waste Treatment Building (Rw/B) is being reconfigured.

Rubble removal work around the Unit 1/2 Rw/B resumed from August 23, as the construction of the course for the rubbleremoving heavy machine was completed in the range for preceding work.

Work to date to remove the SGTS pipes was reviewed and measures to improve reliability by modifying the cutter and others are being examined.



<Rubble removal work>



Unit 2 Trial retrieval of fuel debris

Regarding the robot arm, by conducting mockup tests in Naraha to simulate the actual site, improvements such as modifying the control program are underway to reduce the risk of contact while removing fuel debris.

Moreover, work to install the isolation room started toward opening the X-6 penetration hatch. Damage in the rubber box detected during the work will be addressed.

On this occasion, when the response status based on the test, measures in the site and others were organized, to increase safety and reliability of the trial retrieval (internal investigation and sampling of debris), the process was reviewed to that which added a preparation period for about one or one and a half year and would start the trial retrieval work from around late FY2023.

Unit 2

Commencement of work on ALPS treated water dilution/discharge facility and related facilities

Work on ALPS treated water dilution/discharge facility and related facilities commenced on August 4. Preparation is underway, including laying pipes for the measurement / confirmation facility and the transfer facility around K4 area tanks, building the discharge tunnel by shield machine and installing a partitioning weir in the Unit 5 and 6 intake open channel. Work will continue with

safety prioritized.



Photo taken from inside the shield machine looking from the rear toward the front <Shield machine excavation work>

2/9

Unit 2 Investigation in the control room of the fuel-handling machine

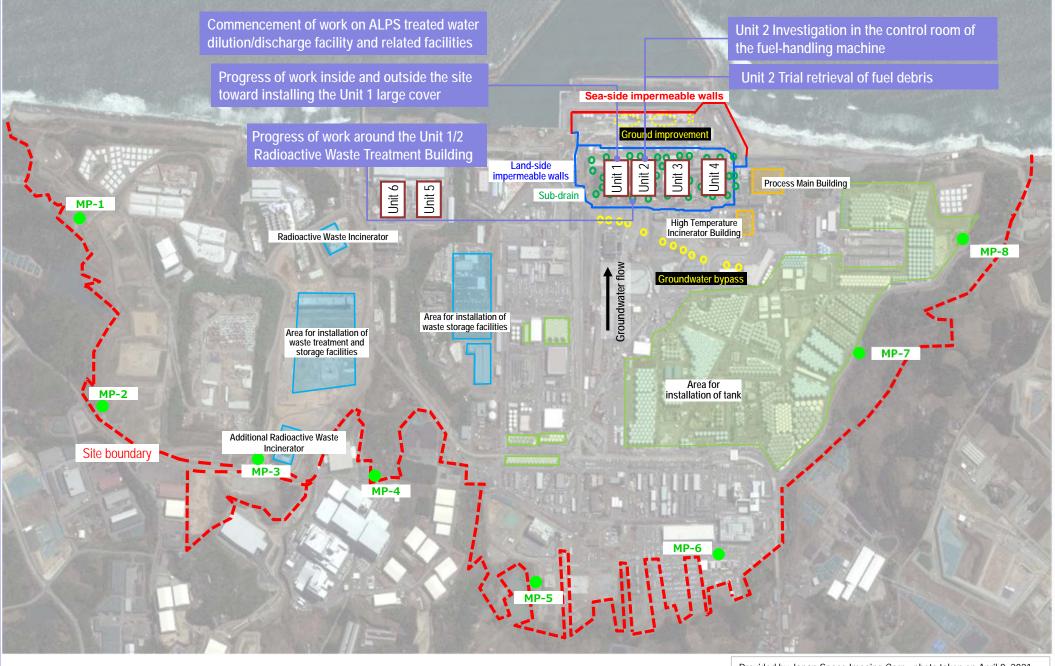
To acquire information to help clarify the accident progress, an investigation, including measuring the dose distribution and the smear paper wipe off, was conducted for the control room of the Unit 2 fuel-handling machine (FHM control room) using the remotely operated robot (SPOT).

On the second floor of the FHM control room. certain places were deemed difficult to investigate by SPOT due to damage detected on the floor surface. For these places, additional investigation is scheduled during work to dismantle the FHM control room, which started from August 22.

Smear samples will be analyzed in the internal <Smear paper wipe off investigation laboratory and the external analysis institute. by the investigation robot (SPOT)>



Major initiatives – Locations on site

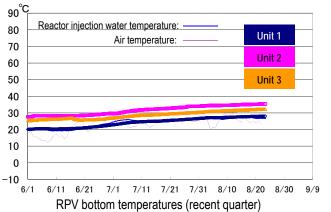


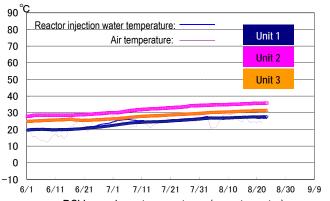
Provided by Japan Space Imaging Corp., photo taken on April 8, 2021 Product (C) [2020] DigitalGlobe, Inc., a Maxar company

I. Confirmation of the reactor conditions

Temperatures inside the reactors

Through continuous reactor cooling by water injection, the temperatures of the Reactor Pressure Vessel (RPV) bottom and the Primary Containment Vessel (PCV) gas phase were maintained within the range of approx. 25 to 40°C for the past month, though it varied depending on the unit and location of the thermometer.

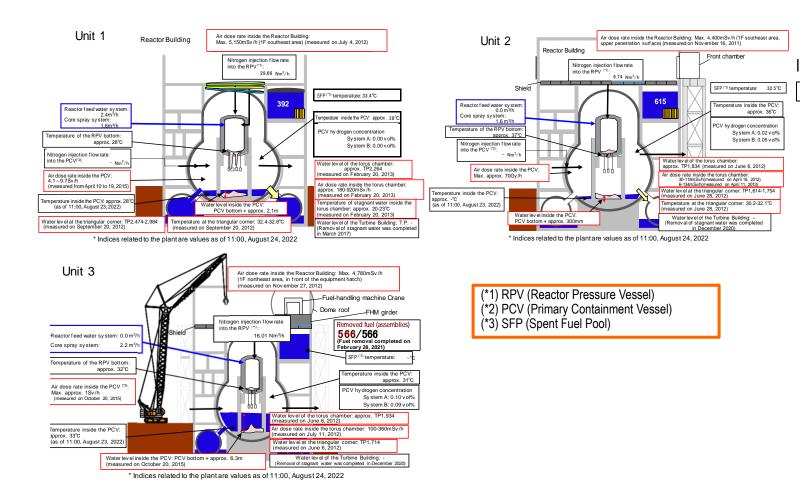




PCV gas phase temperatures (recent guarter)

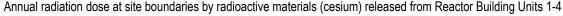
*1 The trend graphs show part of the temperature data measured at multiple points.

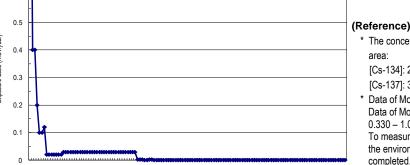
*2 A part of data could not be measured due to maintenance and inspection of the facility and other work



Release of radioactive materials from the Reactor Buildings

As of July 2022, the concentration of radioactive materials newly released from Reactor Building Units 1-4 into the air and measured at the site boundary was evaluated at approx. 2.1×10^{-12} Bg/cm³ and 1.8×10^{-12} Bg/cm³ for Cs-134 and -137 respectively, while the radiation exposure dose due to the release of radioactive materials there was less than 0.00004 mSv/year.





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- Note 1: Different formulas and coefficients were used to evaluate the radiation dose in the facility operation plan and monthly report. The evaluation methods were integrated in September 2012. As the fuel removal from the spent fuel pool (SFP) commenced for Unit 4, the radiation exposure dose from Unit 4 was added to the items subject to evaluation since November 2013. The evaluation has been changed to a method considering the values of continuous dust monitors since FY2015, with data to be evaluated monthly and announced the following month.
- Note 2: Radiation dose was calculated using the evaluation values of release amount from Units 1-4 and Units 5 and 6. The radiation dose of Unit 5 and 6 was evaluated based on expected release amount during operation until September 2019 but the evaluation method was reviewed and changed to calculate based on the actual measurement results of Units 5 and 6 from October.

Other indices

There was no significant change in indices, including the pressure in the PCV and the PCV radioactivity density (Xe-135) for monitoring criticality, nor was any anomaly in the cold shutdown condition or criticality sign detected. Based on the above, it was confirmed that the comprehensive cold shutdown condition had been maintained and the reactors remained in a stabilized condition.

II. Progress status by each plan

Measures for contaminated water and treated water

- Status of contaminated water generated
- Multi-layered measures, including pumping up by sub-drains and land-side impermeable walls, which were buildings.
- After implementing "redirecting" measures (groundwater bypass, sub-drains, land-side impermeable walls and others) contaminated water generated within FY2021 declined to approx. 130 m³/day.
- Measures will continue to further reduce the amount of contaminated water generated.

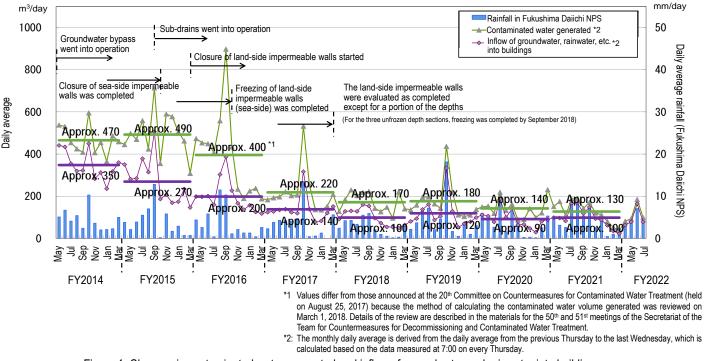


Figure 1: Changes in contaminated water generated and inflow of groundwater and rainwater into buildings

* The concentration limit of radioactive materials in the air outside the surrounding monitoring

[Cs-134]: 2 x 10-5 Bg/cm^{3Marc}

[Cs-137]: 3 x 10-5 Bg/cm3

* Data of Monitoring Posts (MP1-MP8).

Data of Monitoring Posts (MPs) measuring the air dose rate around the site boundary showed 0.330 - 1.042 µSv/h (July 27 - August 23, 2022).

To measure the variation in the air dose rate of MP2-MP8 more accurately, work to improve the environment (trimming trees, removing surface soil and shielding around the MPs) was completed.

implemented to control the continued generation of contaminated water, suppressed the groundwater inflow into

and rainwater prevention measures, including repairing damaged portions of building roofs, the amount of

- Operation of the Water-Treatment Facility special for Sub-drain & Groundwater drains \succ
- At the Water-Treatment Facility Special for Sub-drain & Groundwater drains, release started from September 14, 2015 and up until August 16, 2022, 1,944 release operations had been conducted. The water quality of all temporary storage tanks satisfied the operation target.

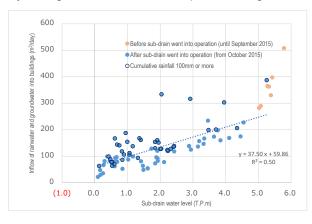
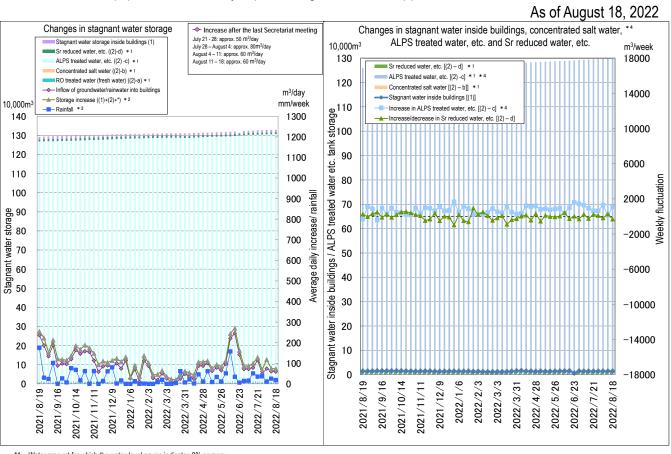


Figure 2: Correlation between inflow such as groundwater and rainwater into buildings and the water level of Units 1-4 sub-drains

- Implementation status of facing \geq
- Facing is a measure involving asphalting the on-site surface to reduce the radiation dose, prevent rainwater infiltrating the ground and reduce the amount of underground water flowing into buildings. As of the end of July 2022, 95% of the planned area (1.450,000 m² on site) had been completed. For the area inside the land-side impermeable walls, implementation proceeds appropriately after constructing a yard from implementable zones that leave the decommissioning work unaffected. As of the end of July 2022, 30% of the planned area (60,000 m²) had been completed.
- Status of the groundwater level around buildings \geq
- The groundwater level in the area inside the land-side impermeable walls has been declining every year due to the land-side impermeable walls and the decline in the set water level of the sub-drains. On the mountain side, the average difference between the inside and outside has been 4-5 m. The water level in the bank area has remained low (T.P. 1.4 m) compared with the ground surface (T.P. 2.5 m).
- As the set water level of the sub-drains declined slightly (T.P. -0.55 \Rightarrow -0.65 m) and others in FY2021, the groundwater level on the sea side of the Unit 1-4 buildings remained low (except during heavy rainfall) compared with the T.P. 2.5 m area.
- Operation of the multi-nuclide removal equipment \geq
- Regarding the multi-nuclide removal equipment (existing), hot tests using radioactive water are ongoing (System A: from March 30, 2013, System B: from June 13, 2013, System C: from September 27, 2013). On March 23, 2022, a pre-service inspection certificate was granted by the Nuclear Regulation Authority and the entire pre-service inspection was completed. The (additional) multi-nuclide removal equipment went into full-scale operation from October 16, 2017. Regarding the (high-performance) multi-nuclide removal equipment, hot tests using radioactive water have been underway (from October 18, 2014).
- As of August 18, 2022, the volumes treated by existing, additional and high-performance multi-nuclide removal equipment were approx. 488,000, 741,000 and 103,000 m³, respectively (including approx. 9,500 m³ stored in the J1(D) tank, which contained water with highly concentrated radioactive materials at the System B outlet of the existing multi-nuclide removal equipment).
- Treatment measures comprising the removal of strontium by cesium-adsorption apparatus (KURION), the secondary cesium-adsorption apparatus (SARRY) and the third cesium-adsorption apparatus (SARRY II) continued. Up until August 18, 2022, approx. 685,000 m³ had been treated.

- Risk reduction of strontium-reduced water \geq
- To reduce the risks of strontium-reduced water, treatment using existing, additional and high-performance multinuclide removal equipment is underway. Up until August 18, 2022, approx. 850,000 m³ had been treated.



*1: Water amount for which the water-level gauge indicates 0% or more To detect storage increases more accurately, the calculation method was reviewed as follows from February 9, 2017: (The revised method was applied from March 1, 2018)

[(Inflow of groundwater/rainwater into buildings) + (other transfer) + (chemical injection into ALPS)] Changed from December 13, 2018 from rainfall in Namie to that within the site

The notation of treated water by the multi-nuclide removal equipment and others was reviewed in accordance with redefining of ALPS treated water by the Government (April 27, 2021 Figure 3: Status of stagnant water storage

- \geq Status of the sea area monitoring related to the handling of ALPS treated water
- The concentration of tritium in seawater within 2km of the port has remained constant for the past year and also remained low at new measurement points within the fluctuation range of seawater in Japan*. The concentration of Cesium-137 increased temporarily, which was considered due to rainfall as in the past fluctuation in seawater around the Fukushima Daiichi Nuclear Power Station. However, it remained constant relative to the measurement value for the past year and at new measurement points and also remained low within the fluctuation range of seawater in Japan*. For tritium, monitoring has been conducted with a lower detection limit since April 18.
- Both concentrations of tritium and Cesium-137 in seawater within 20km of the coast had remained constant for the past year and low within the fluctuation range of seawater in Japan*.
- The concentration of tritium in seawater further than 20km from the coast remained low, including at new measurement points, within the fluctuation range of seawater in Japan*. The concentration of Cesium-137 remained constant for the past year within the fluctuation range of seawater in Japan*.

": The range of the minimum – maxim	num values detected dui
In Japan (including off the coast	of Fukushima Prefectur
Tritium concentration:	0.043 - 20 Bq/L
Cesium-137 concentration:	0.0010 - 0.38 Bq/L
Off the coast of Fukushima Prefe	ecture
Tritium concentration:	0.043 - 0.89 Bq/L
Cesium-137 concentration:	0.0013 - 0.38 Bq/L

rance of the minimum – maximum values detected during April 2018 – March 2020 in the database below re)

Source: Environmental Radioactivity and Radiation in Japan, Environmental Radiation Database https://www.kankyo-hoshano.go.jp/data/database/

- · For the status of fish and seaweed, although no samples were collected in April, measured data is being verified for samples collected in May.
- > Collection status of resin leaking from the Reactor Water Clean-up System spent resin tank room in the Unit 3 Filter Sludge Tank Room (FSTR) building
- On September 1, 2020, leakage of waste liquid and spent resin was detected from pipes connected with the Reactor Water Clean-up System spent resin storage tanks on the basement floor of the Unit 3 FSTR building.
- Spent resin having leaked started to be collected from June 2021 and was then transferred to the waste sludge storage tank (B) of the FSTR building.
- Given the reduced resin collection volume due to the incline inside the tanks, work had been temporarily suspended since March 2022 to review the collection method. After the review, the collection was resumed from August 2022.
- As the collection inside the tanks was completed and spent resin was collected to a level lower than the connection of the leakage pipes, the risk of spent fuel leaking outside the tanks was eliminated. Residual spent fuel outside the tanks will be collected by around the end of November and work will be completed within 2022, including clean up and others.

Fuel removal from the spent fuel pools

Work to help remove spent fuel from the pool is progressing steadily while ensuring seismic capacity and safety.

- Main work to help spent fuel removal at Unit 1
- From late April 2021, work to assemble a temporary gantry and others has been underway in a yard outside the site as part of efforts to install a large cover. The ground assembly was completed for the temporary gantry and lower structure and approx. 40%, for the upper structure.
- A work yard was prepared around the Reactor Building and work to install a large cover started from August 2021.
- From April 13, 2022, drilling to install an anchor in the Reactor Building started. A temporary gantry was also installed from the part where anchors and base plates were installed.
- As preparation to mitigate any dust scattering during work, as well as sprinkling water using cranes, equipment to spray water over the Reactor Building operating floor was installed on the rooftop of the Unit 1 Turbine Building as part of enhanced the measures.
- Main work to help spent fuel removal at Unit 2
- Decontamination to suppress dust scattering on the top floor of the Reactor Building was completed in December 2021 and contamination reduction was confirmed based on smear sampling results. Work to install shielding started from February within a range including above the reactor well, where the dose was observed to peak and will be completed at the end of May.
- From October 28, 2021, ground improvement work started before installing the gantry for fuel removal and was completed on April 19, 2022. Work to install the gantry foundation will then proceed.
- Outside the site, work to prepare a yard for ground assembly of steel frames was completed on March 18. Before the ground assembly, preparation will proceed.

Retrieval of fuel debris

- Progress status toward Unit 1 PCV internal investigation
- To acquire information related to the construction plan to collect deposits and others toward fuel debris retrieval, a remotely operated underwater vehicle (ROV) will be inserted from X-2 penetration into the basement within the PCV to investigate inside and outside the pedestal.
- During June 7-11, the thickness of deposits was measured using the remotely operated submersible ROV-C robot.

- Progress status toward Unit 2 PCV internal investigation and trial retrieval
- The trial retrieval equipment for the Unit 2 fuel debris, which had been developed in the UK, arrived in Japan on July 10, 2021.
- The ongoing performance verification test in a domestic factory (Kobe), which started from August 2021, finished on January 21, 2022.
- The equipment was transported from January 28, 2022 and the robot arm arrived on January 31 and the enclosure, on February 4, at the Naraha Center for Remote Control Technology Development of the Japan Atomic Energy Agency (JAEA) (hereinafter referred to as the "Naraha mockup facility").
- facility.

Plans to store, process and dispose of solid waste and decommission of reactor facilities

Promoting efforts to reduce and store waste generated appropriately and R&D to facilitate adequate and safe storage, processing and disposal of radioactive waste

- > Management status of the rubble and trimmed trees
- As of the end of July 2022, the total storage volume for concrete and metal rubble was approx. 329,900 m³ (+3,200 47,900m³.
- Management status of secondary waste from water treatment
- As of August 4, 2022, the total storage volume of waste sludge was 442 m³ (area-occupation rate: 63%), while that of 85%).
- \triangleright Status of the additional Radioactive Waste Incinerator
- On May 23, the operation of the additional Radioactive Waste Incinerator resumed.
- On June 10, water was detected inside the fly removal system, the incineration operation was suspended for inspection. No external leakage of radioactive materials was identified.
- Equipment to supply water to the system was investigated and no abnormality was detected in any portion. This was assumed to be attributable to frequent starting and stopping of the burner, subsequently resulting in over-supply of the exhaust gas spray following the temperature change.
- Before resuming operation, the temperature control value will be changed and the properties of the ash will be verified.
- On June 18, during a patrol, cracking and others were detected in the seal plate over the joint of the secondary burner and stoker and in the welded part of the rotary kiln joint cylinder.
- As when the cracks were detected, the incineration operation was suspended and negative pressure was maintained by a blower in the area within the affected system, no radioactive material was deemed to have leaked outside.
- After observing the crack surface, the cracking was deemed attributable to ductile fracture by excessive stress, caused by the impact of the Fukushima Offshore Earthquake on March 16. Moreover, cracking of the welded parts due to insufficient strength was also detected.
- Based on occurrence of the above issues, similar facility parts were investigated and new issues, such as loose bolts implemented with restoration targeted for around September.

• From February 14, 2022, the performance verification test and operational training started at the Naraha mockup

m³ compared to the end of June with an area-occupation rate of 88%). The total storage volume of trimmed trees was approx. 129,300 m³ (+100 m³, with an area-occupation rate of 74%). The total storage volume of used protective clothing was approx. 30,400 m³ (-700 m³, with an area-occupation rate of 58%). The increase in rubble was attributable to work around Units 1-4 buildings, construction related to the port, decontamination of flanged tanks and others. As of the end of July 2022, there were ten temporary deposits with storage capacity exceeding 1,000m³, storage

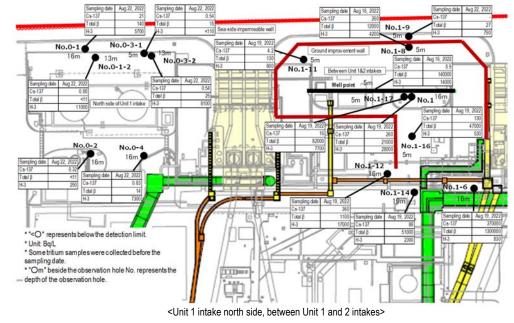
concentrated waste fluid was 9,380 m³ (area-occupation rate: 91%). The total number of stored spent vessels, High-Integrity Containers (HICs) for multi-nuclide removal equipment and other vessels, was 5,402 (area-occupation rate:

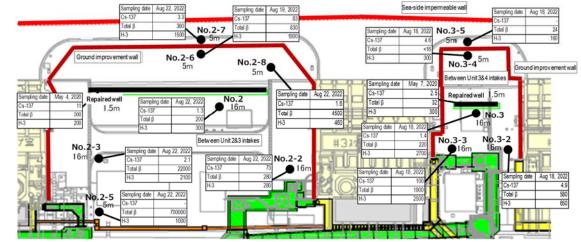
and cracks in furnace refractory materials, were detected. Parts of these issues are being repaired and other treatment,

Reduction in radiation dose and mitigation of contamination

Effective dose-reduction at site boundaries and purification of port water to mitigate the impact of radiation on the external environment

- Status of groundwater and seawater on the east side of Turbine Building Units 1-4
- In the Unit 1 intake north side area, the H-3 concentration was below the legal discharge limit of 60,000 Bg/L at all observation holes and remained constant or has been declining overall. The concentration of total β radioactive materials has remained constant overall but increased temporarily from April 2020 and is even increasing or declining at many observation holes at present, including Nos. 0-1-2, 0-3-1, 0-3-2 and 0-4. The trend continues to be monitored carefully.
- In the area between the Unit 1 and 2 intakes, the H-3 concentration has remained below the legal discharge limit of 60,000 Bg/L at all observation holes. It has been increasing or declining at Nos. 1-14 and 1-17 but has otherwise remained constant or been declining overall. The concentration of total ß radioactive materials has remained constant overall but been increasing or declining at many observation holes, including Nos. 1-6, 1-9, 1-11, 1-12, 1-14, 1-16 and 1-17. The trend continues to be monitored carefully.
- In the area between the Unit 2 and 3 intakes, the H-3 concentration has been below the legal discharge limit of 60,000 Bq/L at all observation holes. It has been increasing and declining at Nos. 2-3, 2-5, 2-6 and 2-7 but has remained constant overall. The concentration of total β radioactive materials has remained constant overall but been increasing or declining at Nos. 2-3, 2-5 and 2-6. The trend continues to be monitored carefully.
- In the area between the Unit 3 and 4 intakes, the H-3 concentration has been below the legal discharge limit of 60,000 Bg/L at all observation holes. It has remained constant or been declining overall. The concentration of total β radioactive materials has remained constant overall but has been increasing or declining at many observation holes, including Nos. 3-4 and 3-5. The trend continues to be monitored carefully.
- In the groundwater on the east side of the Turbine Buildings, as with the total β radioactive materials, the concentration of cesium has also remained constant but been increasing or declining and exceeded the previous highest record at some observation holes. Investigations into fluctuation are underway for Nos. 0-3-2, 1, 1-6, 2-6 and 3-3.
- The concentration of radioactive materials in drainage channels has remained constant overall, despite increasing during rainfall.
- In the open channel area of seawater intake for Units 1 to 4, the concentration of radioactive materials in seawater has remained below the legal discharge limit and been declining long term, despite temporary increases in Cs-137 and Sr-90 noted during rainfall. They have also been declining following the completed installation and the connection of steel pipe sheet piles for the sea-side impermeable walls. The concentration of Cs-137 has remained slightly higher in front of the south side impermeable walls and slightly lower on the north side of the east breakwater since March 20, 2019, when the silt fence was transferred to the center of the open channel due to mega float-related construction.
- In the port area, the concentration of radioactive materials in seawater has remained below the legal discharge limit and been declining long term, despite temporary increases in Cs-137 and Sr-90 observed during rainfall. They have remained below the level of those in the Units 1-4 intake open channel area and been declining following the completed installation and connection of steel pipe sheet piles for the sea-side impermeable walls.
- In the area outside the port, regarding the concentration of radioactive materials in seawater, those of Cs-137 and Sr-90 declined and remained low after steel pipe sheet piles for the sea-side impermeable walls were installed and connected. Regarding the concentration of Cs-137, a temporary increase was sometimes observed on the north side of the Unit 5 and 6 outlets and near the south outlet due to the influence of weather, marine meteorology and other factors. Regarding the concentration of Sr-90, variation has been observed in FY2021 in the area outside the port (north and south outlets). Monitoring of the tendency continues, including the potential influence of the weather, marine meteorology and others.





<Between Unit 2 and 3 intakes, between Unit 3 and 4 intakes>

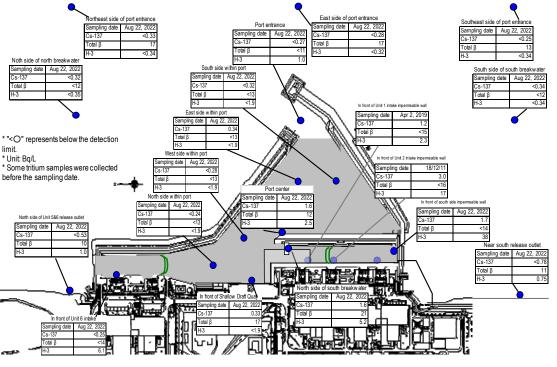


Figure 5: Seawater concentration around the port

Figure 4: Groundwater concentration on the Turbine Building east side

Outlook of the number of staff required and efforts to improve the labor environment and conditions

Adequate number of staff will be secured in the long-term, while firmly implementing radiation control of workers. The work environment and labor conditions will be continuously improved by responding to the needs on the site.

- Staff management
- The monthly average total of personnel registered for at least one day per month to work on site during the past quarter from April to June 2022 was approx. 9,100 (cooperating company workers and TEPCO HD employees), which exceeded the monthly average workforce (approx. 6,800). Accordingly, sufficient personnel are registered to work on site.
- It was confirmed with the prime contractors that the estimated manpower necessary for the work in September 2022 (approx. 4,000 workers per day: cooperating company workers and TEPCO HD employees) would be secured at present. The average numbers of workers per day for each month (actual values) for the most recent 2 years were maintained, with approx. 3,000 to 4,200.
- The number of workers from within Fukushima Prefecture remained constant and those outside, decreased slightly. The local employment ratio (cooperating company workers and TEPCO HD employees) as of July 2022 remained constant at around 70%.
- The average exposure doses of workers were approx. 2.54 and 2.60 and 2.51 mSv/person-year during FY2019, 2020 and 2021, respectively. (The legal exposure dose limits are 100 mSv/person and 50 mSv/person-year over five years, the TEPCO HD management target is 20 mSv/person-year).
- For most workers, the exposure dose was sufficiently within the limit and allowed them to continue engaging in radiation work.

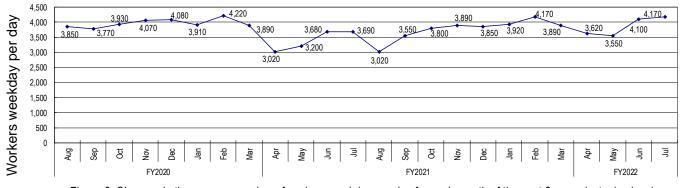
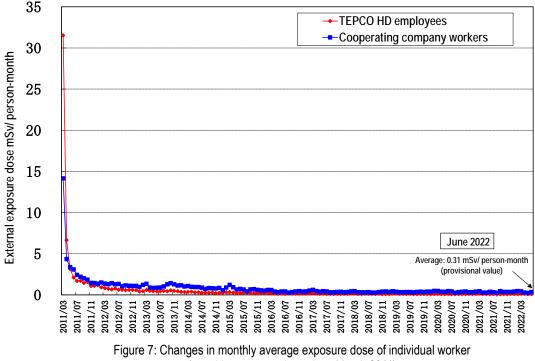


Figure 6: Changes in the average number of workers weekday per day for each month of the past 2 years (actual values)



- Countermeasures to suppress the spread of COVID-19 infections
- for TEPCO HD and cooperating companies:
 - -
 - -
- From the beginning of August, infections have been increasing than ever, mainly among cooperating company
 - these countermeasures (particularly when commuting and taking a break).
 - -
- · The ongoing basic countermeasures to prevent infection spreading, such as requiring employees to take their decommissioning work, prioritizing safety above all.
- As of August 24, 2022, 829 workers (including 114 TEPCO HD employees, 711 cooperating company workers, three July 27)
- No significant influence on decommissioning work, such as a corresponding delay to work processes due to this infection, had been identified.

(monthly exposure dose since March 2011)

Since mid-July, "not to bring" the virus into the nuclear power station, the following additional measures were noted

Employees must check their own physical condition and that of their family members. Those at risk of infection, such as those having moved outside Fukushima Prefecture and their family members having moved from outside, are required to early detect infections by voluntarily undergoing an antigen test.

When commuting, an ongoing occupancy rate of 50% continues to be recommended. Attention is drawn to the need to avoid smoking on board, wear masks, ensure ventilation, refrain from conversation and others. employees. Based on this status, the following countermeasures are implemented to suppress the infection spreading:

Cooperating companies which recorded many infections were visited to inspect the status of the infection suppressing countermeasures, such as ventilation within the office and instruct strict implementation of

Common areas within the site are simultaneously disinfected. Before coming to the company after the Obon holiday, employees must strictly check their own physical condition and voluntarily undergo an antigen test. temperature before coming to the office, wear masks at all times, avoid the "Three Cs" by using the rest house in shifts, eat silently and carefully select business travel, will continue to be properly implemented to proceed with

business partner company employees and one temporary worker) of the Fukushima Daiichi Nuclear Power Station had contracted COVID-19, an increase of 422 workers (including 49 TEPCO HD employees, 372 cooperating company workers and one business partner company employee) from those in the previous published material (as of

- Status of heat stroke cases
- In FY2022, measures to further prevent heat stroke commenced from April to cope with the hottest season.
- FY2022, seven workers suffered heat stroke due to work up until August 22 (in FY2021, seven workers up until the end of August). Continued measures will be taken to prevent heat stroke.

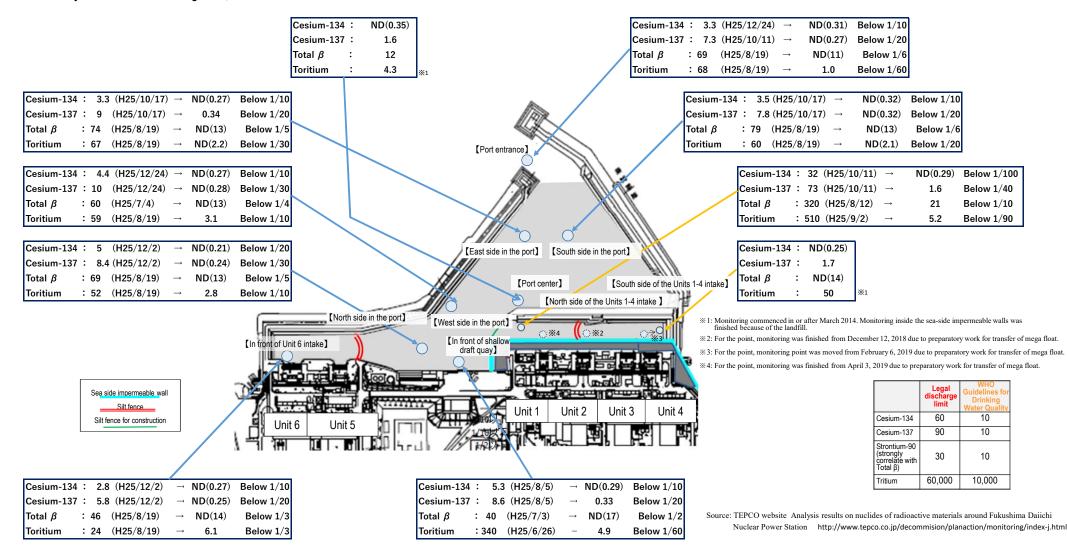
Others

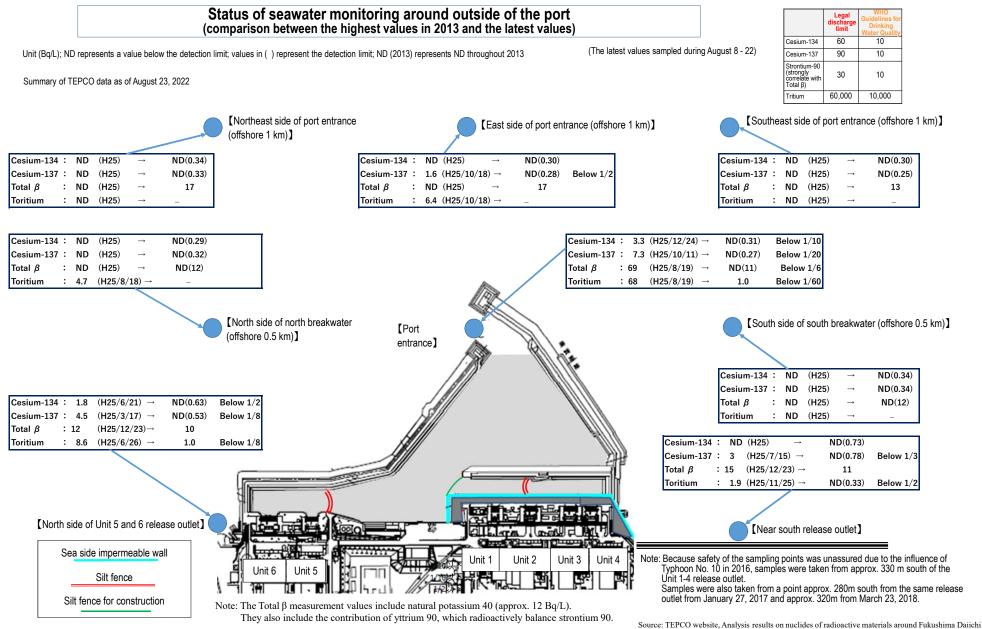
- Progress status of the Mid-and-Long-Term Plan of accident investigation in the Fukushima Daiichi Nuclear Power Station
- As part of efforts to investigate and analyze the accident in the Fukushima Daiichi Nuclear Power Station (hereinafter referred to as 1F), many matters were clarified in the "Internal Accident Report" and "Examination of Unsolved Issues" and others and including instructions provided by internal and external accident investigation committees and others, reflected in the safety measures appropriately. To ensure no recurrence and acquire information to help clarify the whole picture (an in-depth study of the accident progress) and make power reactors even safer, many insights need to be drawn by acquiring on-site information (confirming the actual accident situation) and utilizing and subsequently reflecting these insights in safety measures.
- At the same time, steady decommissioning in 1F is also important. New useful insights for accident investigation and analysis may be acquired in the course of on-site work. However, inadequate data sampling may modify on-site conditions and result in valuable information being lost. The results of the accident investigation and analysis need to be appropriately organized and shared to proceed with on-site work.
- Therefore, to help implement future investigations of the accident in 1F according to plan and substantially by TEPCO HD, the Mid-and-Long-Term Plan of the 1F accident investigations was formulated in November 2021 and is being revised based on the latest work progress and status.

Status of seawater monitoring within the port (comparison between the highest values in 2013 and the latest values)

Note: The Total β measurement values include natural potassium 40 (approx. 12 Bq/L). They also include the contribution of yttrium 90, which radioactively balance strontiur

Summary of TEPCO data as of August 23, 2022

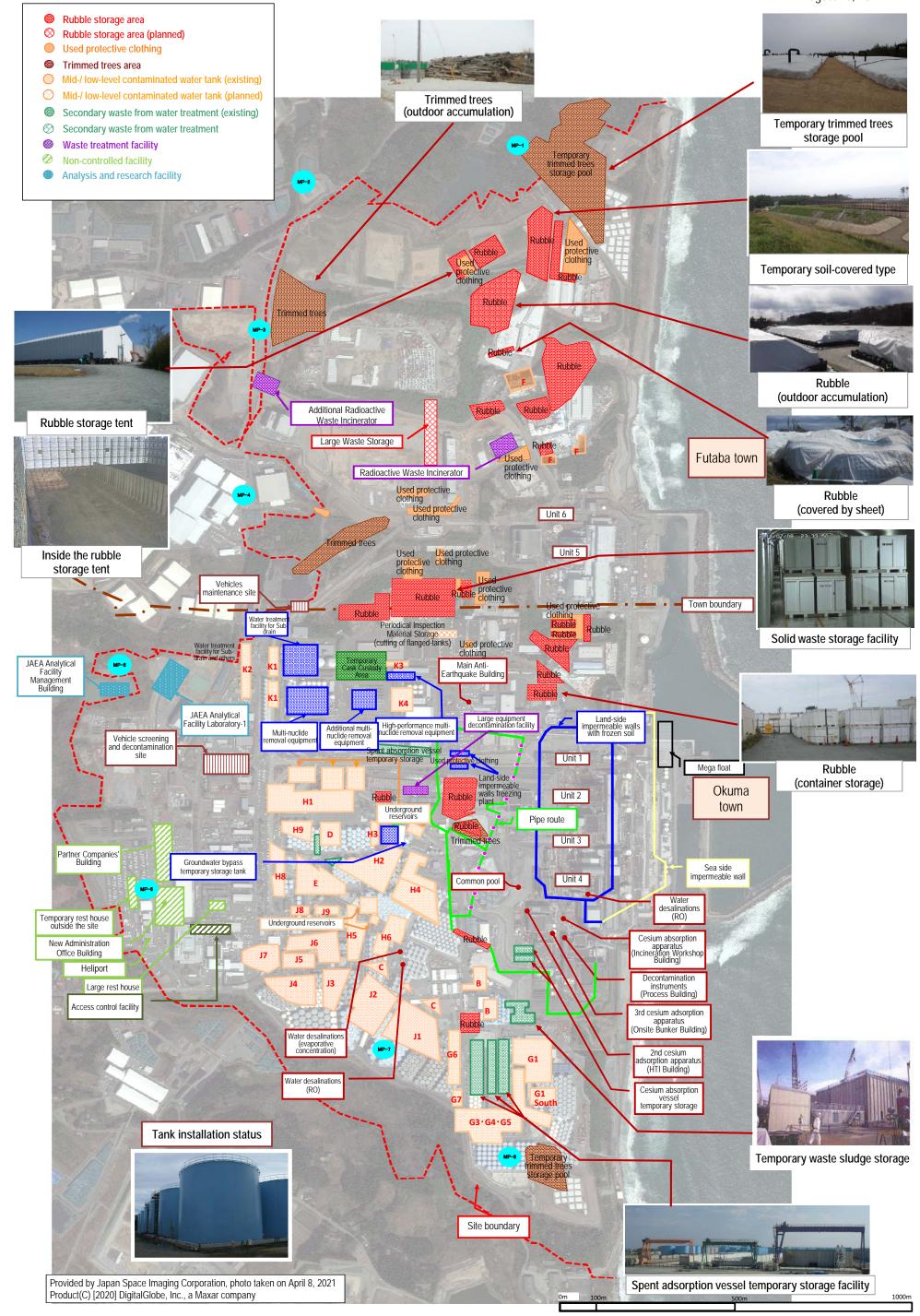




Nuclear Power Station http://www.tepco.co.ip/decommision/planaction/monitoring/index-i.html

TEPCO Holdings Fukushima Daiichi Nuclear Power Station Site Layout

Appendix 2 August 25, 2022



1-1 Contaminated water management

- Efforts to promote contaminated water management based on three basic policies:
 - ① "Remove" the source of water contamination ② "Redirect" fresh water from contaminated areas
 - ③ "Retain" contaminated water from leakage

Milestones of the Mid- and-Long-Term Roadmap (major target processes)

• [Completed] Suppressing the amount of contaminated water generated to 150 m3/day or less (within 2020)

• Suppressing the amount of contaminated water generated to 100 m3/day or less (within 2025)

Reference August 25, 2022 Secretariat of the Team for Countermeasures for Decommissioning, Contaminated Water and Treated Water 1/6

		2011 2012	2013	2014	2015	2016	2017	2018	2019	2020	2021	2022
	1	2 3 4 5 6 7 8 9 13 11 12 1 2 3 4 5 6 7 8 9 10 11 1 2011 2012	2 1 2 3 4 5 6 7 8 9 10 11 12 2013	2014	2 1 2 3 4 5 6 7 8 9 10 11 2015	2016	2017	2018	2 1 2 3 4 5 6 7 8 9 10 11 2019	12 1 2 3 4 5 6 7 8 9 10 11 12 2020	2 2 4 5 6 7 8 9 10 11	12 1 2 3 4 5 6 7 8 9 10 11 2022
		Z011 2012 Z012 Reception start of contaminated water to Central Waste Treatment		2014	2015	2016	2017	2018	2019	2020	2021	2022
						densed salt water complete		⊽Pu	ification of strontium-reduced water in		rontium-reduced water complete	
			ZAL									
			Cer	sium Adsorption Apparatus (KURION)		Adsorption Apparatus (KURION) (from	2015.1.6)					
	Contaminated water			~	Reduction of strontium by 2nd Cesiur	n Adsorption Apparatus (SARRY) (from	2014.12.26)					
	treatment facility									trontium by 3rd Cesium Adsorption Appara	tus (SARRY II) (from 2019.7.12)	
			Multi-nuclide Removal Fauin	ment (ALPS) (System A: from 2013.3.		um-reduced water (ALPS: from 2015.12 n C: from 2013.9.27, hot tests conducted	4, additional: from 2015.5.27, high-perf	ormance: from 2015.4.15)				
					lide Removal Equipment (additional A			f full-scale operation (from 2017.10.16)				4
Contaminated water management			Mult	ti-nuclide removal	uclide Removal Equipment (high per	formance ALPS) (from 2014.10.18, hot to	ests conducted)	m³/day 1000	 Sub-drains went into 	encretion	Rainfall in Fukushima Daii	ichi NPS
[Remove]		Landing of 2 nd Ces Adsorption Apparatus (sium equ	uipment (ALPS)				Gr , we	oundwater bypass	and-side impermeable walls started	-II-Contaminated water gener 	water, etc.
		, tooption reparately (⊽Tre	nch Purification by mobile equipment	· · · · · · · · · · · · · · · · · · ·	Completion of tunnel filling		800		7		40 %
			Unit 2			nt water complete		s. wa	Is was completed	Freezing of land-side The land-side impermetes walls were evaluated as corrected except for a portion of	pleted	gerain
							Unit 2 seawater Shaft D filli	pipe trencn	оргок. 470Аррсок. 490	(For the three unificzen depth	sections, freezing was completed by September 2018	30 mainte
ļ	Removal of					gnant water complete of shaft filling (except for upper part of		400		ox. 400	t.	20
	from seawater pipe	[Removal of contaminated wat	ter in -	U	nit 3			A	opfox 350	Approx. 220	Â.	aDaio
	trench	seawater pipe trench]	1			Ill complete		200	Approx 270	Approx. 170	Approx. 180 Approx. 140 Ap	ADTOX. 130
					7	✓Transfer stagnant water complete			Անտնոնի հետնվել	Approx-140 Approx-160	Approx 120 Approx 90 A	oprox 100
				U	nit 4	Completion of filling parts running ove I I I	AND IN COMPANY OF THE OWNER	ι di	Ner Parka Son and Angel and Ang	Nar Nar Nar Nar Nar Nar Nar Nar Nar Nar	Ju Mary Mary Mary Mary Mary Mary Mary Mary	Jul Jul
							Company MALLING		Y2014 FY2015 FY20	16 FY2017 FY2018		Y2021 FY2022
ļ											ressing the average amount of minated water generated to	,
	Groundwater bypass	⊽Installa	ation start of groundwater bypass	Operation start operation	I f groundwater bypass (draina	ge started from 2014.5.21)					ox. 130 m ³ /day	
	Sub-drain			art of Water-Treatment Facility special n & Groundwater drains	2 0par	tion start of sub-drain (drainag	a started from 2015 0 14)	▼Enhancement of tree	atmost consolity			
Contaminated water management			for Sub-drain	a Groundwater drains		capacity: 1000 m3/day)	e starteu iroin 2015.9.14)	(2000 m3/day)				
[Redirect]			-				T2Ptart of maintenance o	peration on north and south sides				
	Land-side impermeable well					V Start of maintenance of						
						Transie stat		Transie a completion (in all sections		
				V Installation start of	and-side impermeable walls		Start of maintenance operation ⊽	▼Freezing completion (except for some parts)			
							(feeine)					
	Facing					Completion of waterproof pa (except for areas of 2.5 and))	l 6.5m above sea level and around Uni	t 1-4)		Completion of waterproof paver (except for around Unit 1-4)	nent (lacing)	
		High concentration of radio detected from observa	oactive materials ⊽ ⊽Area 2.5m above ation well of bank	e sea level – Start of ground improve	ment by water glass							
				ing of water from contaminated areas	(well point)							
	Bank groundwater measures	⊽Installation start of seaside	e impermeable walls		⊽lns	tallation of seaside impermeabl	e walls complete					
						1						
					\\ \\ \\ \\ \\ \\ \\ \\ \\ \\ \\ \\ \\	peration start of groundwater drain (pum	ping-up started on 2015.11.5)					
					TDuifation of DO an	ndensed salt water complete						
		⊘Storage in steel square tanks		⊽Re	placement of steel square tanks com					iks complete (except for condensed waste	liquid storage tank)	
Contaminated water	Storage facility		\	e (300L) from flanged tank								
management												
(Retain)		Storage in flanged cylindrical tanks		Completion of fence to prevent leakag				∇P		ed water in flanged tanks complet	9	
					e neigni complete				✓ iranster and storage of all t	reated water in welded-joint tanks		
			VI opkozo of contrainent d	unter from underground machine	Short of Impofes to Imples							
			✓ Leakage of contaminated	water from underground reservoir => minated water to tanks complete	Start OF LINES IN LONGING							
											rontium-reduced water complete	
			Storage in cylindrical stee	i weided-joint anks						∨Puntcation of s	romunti-reduced water complete	
					water within tank fences by rainwater tr	eatment facility (from 2014 5 21)						T
			1	v opnining sidt of failt	indian mailt blink forfood by idilfwdibl u	outron, doing (nom 2014.0.21)	1			1		



Legend



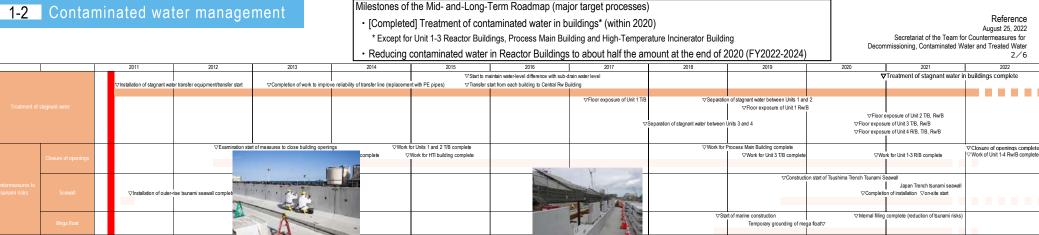












Chishima Trench Tsunami Seawall complete Construction of Japan Trench Tsunami Seawall

2 Handling of ALPS treated water

Tritiated Water Taskforce (2013.12 - 2016.5, 15 meetings)

Tank area viewed from the Large Rest House (2015.10.29)

2015

Multi-nuclide ALPS treated

Removal

Equipment

[ALPS]

water, etc.

Sample tank of multi-

storage tank

2014

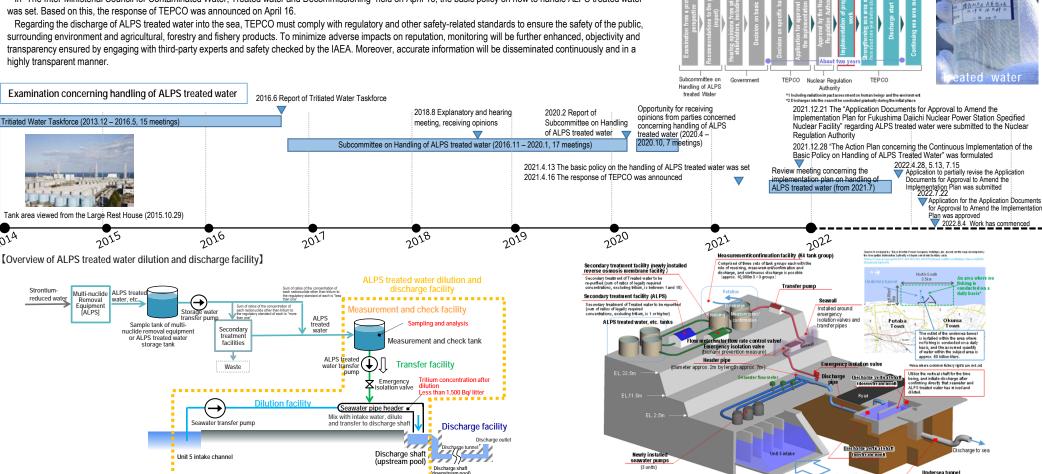
Strontium

reduced water

In "The Inter-Ministerial Council for Contaminated Water, Treated water and Decommissioning" held on April 13, the basic policy on how to handle ALPS treated water was set. Based on this, the response of TEPCO was announced on April 16.

Regarding the discharge of ALPS treated water into the sea, TEPCO must comply with regulatory and other safety-related standards to ensure the safety of the public. surrounding environment and agricultural, forestry and fishery products. To minimize adverse impacts on reputation, monitoring will be further enhanced, objectivity and transparency ensured by engaging with third-party experts and safety checked by the IAEA. Moreover, accurate information will be disseminated continuously and in a highly transparent manner.

..............................



Set in "The Inter-Ministerial Council for

Contaminated Water, Treated Water and

ecommissioning issues" held on April 13.

December 21

2021

July 22,

2022

August 4,

2022

Undersea tunnel

(approx.1km)

eawater used for dilution

(intake from outside the harbor)

3 Removal of fuel from spent pool

Milestones of the Mid- and-Long-Term Roadmap (major target processes)

Completion of Unit 1-6 fuel removal (within 2031)

Completion of installation of Unit 1 large cover (around FY2023), start of Unit 1 fuel removal (FY2027-2028)

Reference

3/6

August 25, 2022

Secretariat of the Team for Countermeasures for

Decommissioning, Contaminated Water and Treated Water

Start of Unit 2 fuel removal (FY2024-2026)

Legend Storage and handling of fuel 2011 2014 2018 2020 2021 2012 2013 2016 2017 2019 2015 In the Mid- and-Long-Term Roadmap, the Phase 1 target involved starting to remove fuel from inside ▼ 2011.11- 2012.7 Removal of rubble on the Reactor Building top floor All fuel assemblies from Unit 4 had been the spent fuel pool (SFP) of the 1st Unit within two years of completing Step 2 (by December 2013). removed by December 2014. On November 18, 2013, fuel removal from Unit 4, namely the first Unit, got underway and Phase 2 of the ▼ 2012.4-2013.3 Ground improvement and foundation work roadmap ▼ 2013.4-2013.7 Installation of external walls and roof panels started. On November 5, 2014, within a year of commencing fuel removal work, all 1,331 spent fuel assemblies ▼ 2013.6-2013.10 Installation of overhead crane and fuel-handling machine in the pool had been transferred. The transfer of the remaining non-irradiated fuel assemblies to the Unit 6 SFP ▼ 2013.8-2013.10 Removal of rubble inside the reactor well and pool was completed on December 22, 2014. (two of the non-irradiated fuel assemblies were removed in advance in July 2012 for fuel checks) ▼ 2013.11.18 Start of fuel removal <Unit 4 Cover for fuel removal> This marks the completion of fuel removal from the Unit 4 Reactor Building. Fuel removal ▼ 2014.12.22 Fuel removal was completed (1533 assemblies) Unit 4 ▼ 2013.10 Completion of removal of large rubble on the Reactor Building top floor All fuel assemblies from Unit 3 had ▼ 2015.8 Completion of removal of the fuel-handling machine B within the spent fuel pool been removed by February 2021. Overview of the fuel-handling facility inside the cover ▼ 2016.12 Completion of shielding on the Reactor Building top floor <Unit 3 Cover for fuel removal (dome roof) 2019.2.21> Before installing a cover for fuel removal, the Fuel-handling machine 2017.1 Installation start of a cover for fuel removal process of removing large rubble from the spent fuel ▼ 2019 4 15 Start of fuel removal pool was completed in November 2015. To ensure 2021.2.28 Fuel removal completed (566 assemblies) safe and steady fuel removal, training via remote control was conducted at the factory using the actual Unit 3 On-site transportation fuel-handling machine to be installed on site (February - December 2015). Installation of the fuel removal cover was completed on February 23, 2018. With fuel removal in mind, rubble retrieval training ▼ 2015.3-2016.11 Yard construction inside the pool, which was scheduled in conjunction with fuel removal training, started from March 15. Unit 2 ▼ 2016.9-2017.4 West-side gantry installation work 2019 and fuel removal started from April 15, 2019. Overview of fuel removal ▼ 2017.5 Opening a hole in the west-side external wall Fuel removal was completed on February 28, 2021. (bird's-eve view) ▼ 2018.8-2020.12 Moving and containment of remaining objects For Unit 2, with the removal of spent fuel in mind, a "gantry for fuel removal" (gantry and front room) will be constructed on the south side of the building. ▼ 2020.6 Investigation inside the spent fuel pool ▼ 2021.10-2022.4 Ground improvement work Jnit 2 Construction of gantry for fuel removal> As part of efforts to remove fuel from the Unit 2 spent fuel pool and based on findings from Unit 2 internal operating floor investigations from November 2018 to February 2019, instead of fully dismantling the upper part of the building, the decision was made to install a small opening on ▼ 2020.3-6 Installation of spent fuel pool cover the south side and use a boom crane. Examination continues to initiate fuel removal from ▼ 2020.9-11 Measures to prevent and alleviate rubble falling EY2024 to EY2026 ▼ 2020.11-2021.6 Dismantling of remaining cover ✓ 2017.12 Completion of building cover dismantling and windbreak fence installation ✓ 2018.1-2020.12 Rubble removal on the north side of Reactor Building 20 <Reference> Progress to date ▼ 2021.8 Start of large cover pre-work For Unit 1, a large cover will be installed over the whole Previously, scope to recover the existing overhead crane ▼ 2018.9-12 Removal of X-braces building, within which rubble will be removed. and the fuel-handling machine was examined. However the high radiation dose inside the operating floor meant Unit 1 the decision was taken to dismantle the upper part of the As part of efforts to remove fuel from the Unit 1 spent fuel pool, investigations are underway to building in November 2015. Findings from internal ascertain the conditions of the fallen roof on the south side and the contamination of the well plug. investigations of the operating floor from November 2018 Based on the results, "the method initially installing a large cover over the Reactor Building, then to February 2019 underlined the potential to conduct removing rubble within the cover" was selected to ensure safer and more secure removal. Work to install limited work there and the means of accessing from the a large cover started from August 2021. Work to complete the installation of a large cover by around south side was examined. FY2023 is ongoing, with fuel removal scheduled to run from FY2027 to FY2028. <Reference> Progress to date Rubble removal on the north side of the operating floor started from January 2018 and has been implemented sequentially. In July and August 2019, the well plug, which was misaligned, was investigated, followed in August and September by the conditions of the overhead crane. Based on the results of these investigations, as the removal requires more careful work taking dust scattering into consideration, two methods were examined: Installing a cover after rubble removal, initially installing a large cover over the Reactor Building, then removing rubble inside the cover. <Unit 1 Dismantling of remaining cover> Rubble removal (image Fuel removal (image) 2020 2013 2016 2017 2018 2019 2021 2011 2012 2014 2015 * Part of the photo is corrected because it includes machine information related to nuclear material prote

Milestones of the Mid- and-Long-Term Roadmap (major target processes)

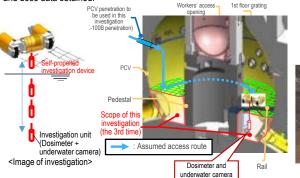
Start of fuel debris retrieval from the first unit (Unit 2). Expanding the scale in stages (within 2021 * The schedule will be extended for about 1 year due to the spread of COVID-19 infections)

Before removing fuel debris, investigations inside the Primary Containment Vessel (PCV) are conducted to inspect the conditions there, including locations of fuel debris.

Unit 1 Investigation overview

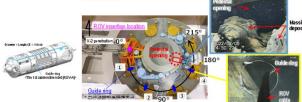
 In April 2015, a device having entered the inside of the PCV via a narrow opening (bore: 0100 mm) collected information such as images and airborne dose inside the PCV 1st floor.

• In March 2017, an investigation using a self-propelled investigation device was conducted to inspect the spreading of debris to the basement floor outside the pedestal, with images taken of the PCV bottom status for the first time. The conditions inside the PCV will continue to be examined, based on the imagery and dose data obtained.



In February, the first remotely operated underwater vehicle (ROV-A) was inserted to install "guide rings" which will facilitate the investigation. As installation of guide rings has been completed, then a detailed investigation will be implemented.

In this investigation, distribution of deposits outside the pedestal and their characteristics or others will also be investigated. The results of these investigations will be utilized in the examination of method and procedures toward future debris retrieval.



Unit 1 PCV internal investigation

Investigations inside the PCV	1st (2012.10)	Acquiring images Measuring the air temperature and dose rate Measuring the water level and temperature Sampling stagnant water Installing permanent monitoring instrumentation			
	2nd (2015.4)	Confirming the status of the PCV 1st floor - Acquiring images - Measuring the air temperature and dose rate - Replacing permanent monitoring instrumentation			
	3rd (2017.3)	Confirming the status of the PCV 1st basement floor - Acquiring images - Measuring the dose rate - Sampling deposit - Replacing permanent monitoring instrumentation			
Leakage points from PCV	- PCV vent pipe vacuum break line bellows (identified in 2014.5) - Sand cushion drain line (identified in 2013.11)				
Evaluation of the location of fuel debris inside the reactor by measurement using muons					

Confirmed that there was no large fuel in the reactor core. (2015.2-5)

Unit 2 Investigation overview

 In January 2017, a camera was inserted from the PCV penetration to inspect the conditions of the rail on which the robot traveled. The results of a series of investigations confirmed some gratings had fallen and deformed as well as a quantity of deposit inside the pedestal.

 In January 2018, the conditions below the platform inside the pedestal were investigated Based on the analytical results of images obtained in the investigation, deposits, probably including fuel debris, were found at the bottom of the pedestal. Moreover, multiple parts exceeding the surrounding deposits were also detected. We presumed that there were multiple instances of fuel debris falling.

· In February 2019, an investigation touching the deposits at the bottom of the pedestal and on the platform was conducted and confirmed that the pebble-shaped deposits, etc. could be moved and that hard rock-like deposits that could not be gripped may exist.

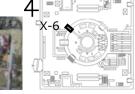




Bottom of the pedestal (after being processed in panoramic image visualization)

· In October 2020, as part of work to prepare for the PCV internal investigation and trial retrieval, a contact investigation to study deposits inside the penetration (X-6 penetration) was conducted, which involved inserting a guide pipe incorporating an investigative unit into the penetration. This confirmed that deposits inside the penetration had not deformed and come unstuck. The investigative information obtained will be utilized in the mockup test of the equipment to remove deposits inside the X-6 penetration.





<Work in front of the penetration>

<Unit 2 Reactor Building 1st floor

Location of the penetration>

<status (february="" 9)<="" inside="" pcv="" th="" the=""><th>•)></th><th>Unit 2 PCV i</th><th>nternal investigation</th><th></th></status>	•)>	Unit 2 PCV i	nternal investigation		
ture and dose rate and temperature oring instrumentation			1st (2012.1)	- Acquiring images - Measuring the air temperature	
			2nd (2012.3)	- Confirming water surface - Measuring the water temperature - Measuring the dose rate	
PCV 1st floor	inside th PCV	Investigations inside the	3rd (2013.2 - 2014.6)	 Acquiring images - Sampling stagnant water Measuring water level - Installing permanent monitoring instrumentation 	
ture and dose rate itoring instrumentation		PCV	4th (2017.1-2)	- Acquiring images - Measuring the dose rate - Measuring the air temperature	
PCV 1st basement floor				5th (2018.1)	- Acquiring images - Measuring the dose rate - Measuring the air temperature
1			6th (2019.2)	- Acquiring images - Measuring the dose rate - Measuring the air temperature - Determining characteristics of a portion of deposit	
2014.5)		Leakage points from PCV	- No leakage from the torus chamber rooftop	- No leakage from any internal/external surfaces of S/C	
	Evaluation of the location of fuel debris inside the reactor by measurement using muons				

Εv and outer periphery of the reactor core. It was assumed that a significant portion of fuel debris existed at the bottom of RPV. (2016.3-7)

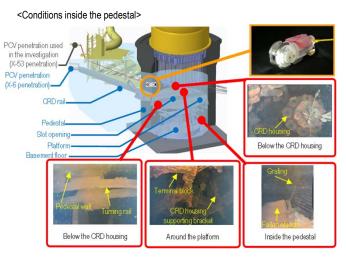
Unit 3 Investigation overview

• In October 2014, the conditions of X-53 penetration, which may be under water and which is scheduled for use to investigate the inside of the PCV, was investigated via remote-controlled ultrasonic test equipment. The results showed that the penetration was not under water.

 In October 2015, to confirm the conditions inside the PCV, an investigative device was inserted into the PCV from X-53 penetration to obtain images, data on dosage and temperature and sample stagnant water. No damage to the structure and walls inside the PCV was identified and the water level was almost identical to estimated values. In addition, the dose inside the PCV was confirmed to be lower than in other Units.

• In July 2017, the inside of the PCV was investigated using the underwater ROV (remotely operated underwater vehicle) to inspect the inside of the pedestal. Analysis of the imagery obtained in the investigation identified damage to multiple structures and the supposed core internals.

· Videos obtained in the investigation were reproduced in 3D. Based on the reproduced images, the relative positions of the structures, such as the rotating platform slipping off the rail with a portion buried in deposits, were visually understood.

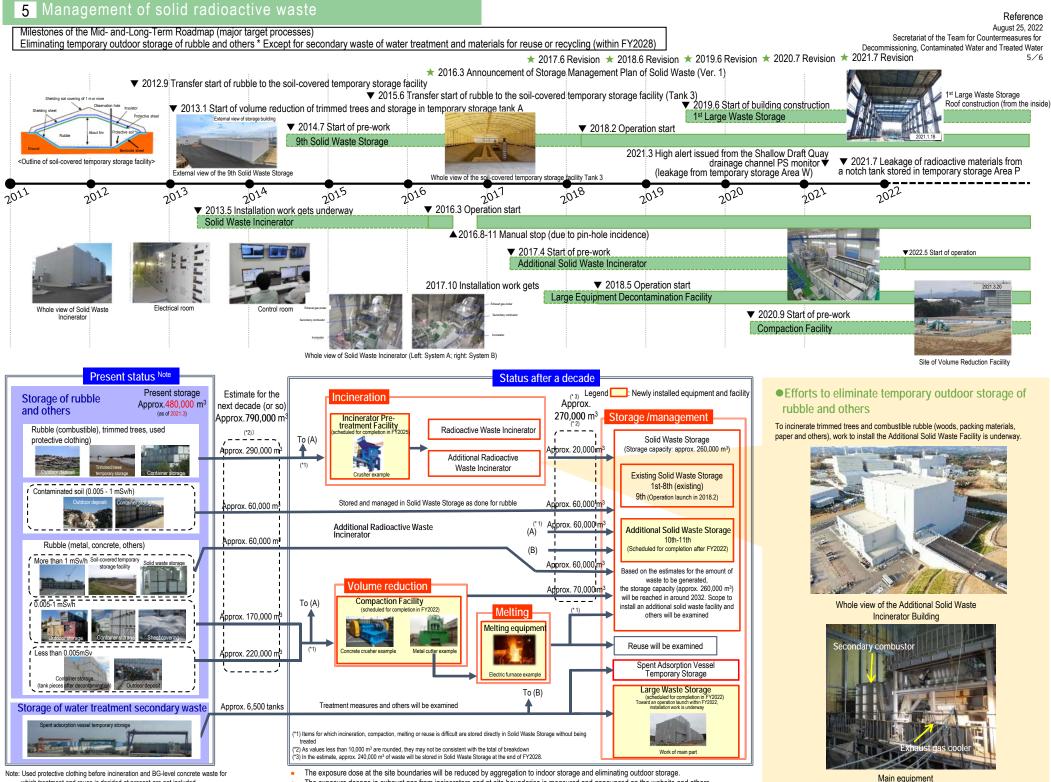


Unit 3 PCV internal investigation

Investigations inside the PCV	1st (2015.10-12)	- Acquiring images - Measuring the air temperature and dose rate - Measuring the water level and temperature - Sampling stagnant water - Installing permanent monitoring instrumentation (2015.12)			
	2nd (2017.7)	 Acquiring images Installing permanent monitoring instrumentation (2017.8) 			
Leakage points from PCV					
Evaluation of the location of fuel debris inside the reactor by measurement using muons. The evaluation confirmed that no large lump existed in the core area where fuel had been placed and that a portion of the fuel debris potentially existed at the bottom of the RPV. (2017.5-9)					

Images are provided by the International Research Institute for Nuclear Decommissioning (IRID)

Reference August 25, 2022 Secretariat of the Team for Countermeasures for Decommissioning, Contaminated Water and Treated Water 4/6

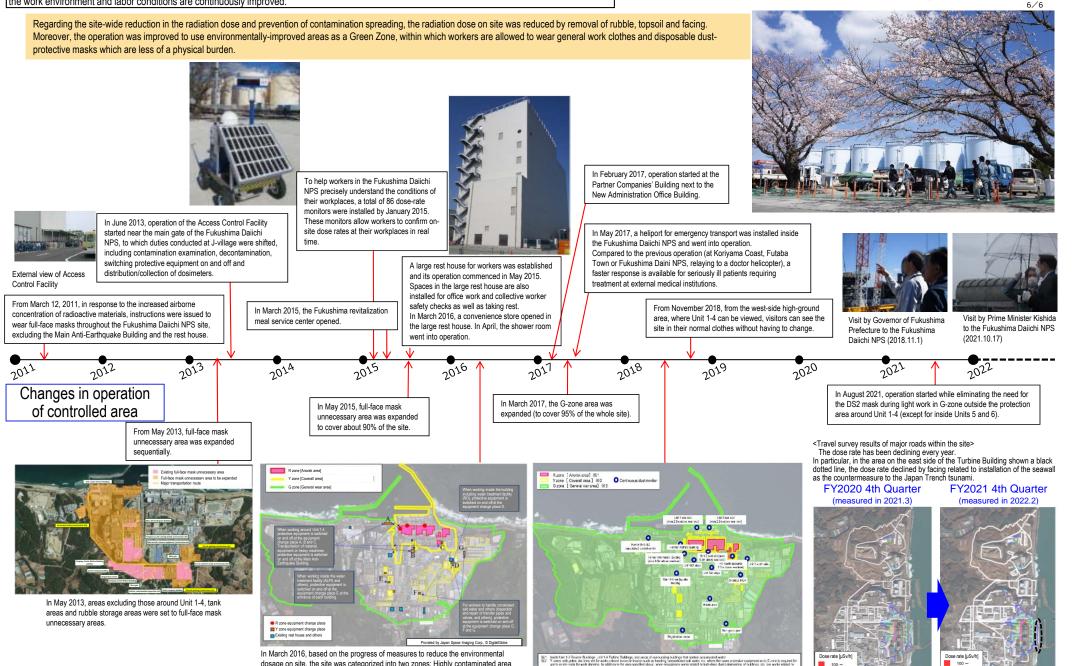


which treatment and reuse is decided at present are not included

The exposure dosage in exhaust gas from incinerators and at site boundaries is measured and announced on the website and others.

While ensuring reliable exposure dose management for workers, sufficient personnel are secured. Moreover, while getting a handle on on-site needs, the work environment and labor conditions are continuously improved.

Reference August 25, 2022 Secretariat of the Team for Countermeasures for Decommissioning, Contaminated Water and Treated Water 6/6



dosage on site, the site was categorized into two zones: Highly contaminated area around Unit 1-4 buildings, etc. and other areas where limited operation started to optimize protective equipment according to each category.

In May 2018, within about 96% of the site, workers are allowed to wear light equipment such as general workwear and disposable dust-protective masks.

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Provided by Japan Space Imaging Corp., © DigitalGlobe

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